

No. 210

June 1982

You can observe a lot just by watching — Yogi Berra

Sixth ICRS Reminder

The deadline for submitting papers for the 6th International Conference on Radiation Shielding is September 6, 1982. Persons wishing to submit papers for the May 16-20, 1983 meeting should prepare and send summaries as soon as possible. Instructions for preparation of summaries are found in the CALL FOR PAPERS given in *RSIC Newsletter* No. 208, April 1982.

The conference, to be held in Tokyo, Japan, is under the General Chairmanship of *Hiroshi Ishikawa* (JAERI Director). The Technical Program Chairman, *Takumi Asaoka*, completed an initial round of visits and discussions with sponsoring agencies and committees in April. Details of some of these meetings are summarized in the May 1982 RSIC Newsletter (No. 209).

STANDARDS ACTIONS

We note the following Standards activities.

ANSI – Final Actions

ANSI N13.1-1969 (R1982) (reaffirmation) Sampling Airborne Radioactive Materials in Nuclear Facilities, sets forth the principles that apply in obtaining valid samples of airborne radioactive materials and prescribes acceptable methods and materials for gas and particle sampling. This standard is limited to the collection of samples and does not embrace the measurement of the radioactive materials collected. It is further limited to guides for sampling airborne radioactive materials in installations where work with radioactive materials is conducted, with the primary emphasis on the need to protect the radiation worker. It does include sampling effluent gases prior to or at the point of release to the atmosphere from the installation. Reaffirmation date: March 26, 1982.

PERSONAL ITEM

A. David Rossin has been named the 1982 Electric Industry's Man of the Year. The award was presented on April 27 during the American Power Conference All Engineers Dinner. According to Robert Lincicome, editor of Electric Light and Power Magazine, Rossin was chosen for the honor by the magazine's readers for his "untiring support in the development and implementation of nuclear power in the United States and for appearing in countless public forums to convey the message that nuclear energy is economical and safe."

Rossin, Director of Research for Commonwealth Edison, is on leave and serving as Director of the Nuclear Safety Analysis Center for the Electric Power Research Institute, Palo Alto, California. One of the nation's most active speakers on behalf of nuclear energy, Rossin is a prominent member of the American Nuclear Society. In his role as head of the Nuclear Safety Analysis Center, Rossin monitors safety in nuclear facilities, making recommendations to correct any possible flaws in their design and operation.

CHANGES TO THE RSIC CODES COLLECTION

An update and six new code systems were added to the RSIC computer codes collection during the month of May. Included are two contributions from the Federal Republic of Germany and one each from Australia, Italy, and Japan.

CCC-254/ANISN-ORNL

The CDC 6600/7600 version (C) of this multigroup one-dimensional discrete ordinates transport code system with anisotropic scattering was updated to correct an error in Subroutine PLSNT. Correcting this error eliminates level 8 error messages during compilation. Details of the correction may be requested from RSIC. References: K-1693; NAA-SR-10951; ORNL/TM-4015, ORNL/TM-3049. FORTRAN IV; CDC 6600/7600.

CCC-420/DARTAB

A code system for combining airborne radionuclide environmental exposure data with dosimetric and health effects data to generate tabulations of predicted health impacts was contributed by the Oak Ridge National Laboratory. Options are included to permit the user to request tabulations by various topics (e.g., cancer site, exposure pathway, etc.) to facilitate characterization of the human health impacts of the effluents. Reference: ORNL-5692. FORTRAN IV; IBM 360/370.

PSR-177/NEUPAC

A code package for unfolding neutron spectra data from activation data of dosimeter foils was contributed by Power Reactor and Nuclear Fuel Development Corporation, Tokyo, Japan. Integral quantities and their sensitivities, together with an error estimate of the unfolded spectra and integral quantities are determined. The user may select incore memory or external memory, depending on the number of groups of input data. Reference: PNC N941 80-192Tr. FORTRAN IV; FACOM M-200.

PSR-178/FANAL

A least-squares shape analysis code system was

contributed by Karlsruhe Nuclear Research Center, Federal Republic of Germany, through the Nuclear Energy Agency Data Bank, Gif-sur- Yvette, France. Developed for structural materials (Cr, Fe, Ni ...), FANAL extracts resonance parameters and total cross sections from neutron transmission data obtained by the time-of-flight technique. Doppler broadening is applied to level sequences with vanishing potential scattering (p-wave resonances). Reference: KFK-2129. FORTRAN IV; IBM 3033 and 370/168.

PSR-179/FANAC

A shape analysis code system for resonance parameter extraction from neutron capture data was contributed by Karlsruhe Nuclear Research Center, Federal Republic of Germany, through the Nuclear Energy Agency Data Bank, Gif-sur-Yvette, France. Written for light to mediumweight or near-magic target nuclei, FANAC extracts resonance parameters from high-resolution capture vields measured with time-of-flight method. Treating many resonances simultaneously, FANAC permits determination of up to 20 cross section parameters by simultaneously fitting calculated capture yield curves to experimental data from up to five time-of-flight measurements that may differ with respect to sample thickness, flight path or other experimental characteristics. Reference: KFK-2145. FORTRAN IV; IBM 3033 and 370/168.

PSR-182/XLACS-IIA

A code system for producing weighted multigroup neutron cross sections from ENDF/B data was contributed by the Australian Atomic Energy Commission, Sutherland, Australia. Based on XLACS-II (packaged in PSR-63), which was developed by the Union Carbide Nuclear Division, Computer Sciences Division at Oak Ridge National Laboratory, XLACS-IIA offers an improved method of treating thermal neutron scattering plus extensive changes in resonance treatment. Efficiency is improved in the joining of thermal and epithermal data. Reference: Notes on the AAEC Version of XLACS II. FORTRAN IV; IBM 3031/ 3033.

PSR-183/FOURACES

A group-averaged cross section generator was

contributed by Centro di Calcolo, Comitato Nazionale Energia Nucleare, Bologna, Italy. Designed as a multiformat buffer, FOURACES accepts data in ENDF/B, KEDAK, and UK formats. It computes group-averaged nuclear constants which are flux and temperature dependent. Reference: RT/ FI(73)16. FORTRAN IV; IBM 370/168.

CHANGES TO THE DATA LIBRARY COLLECTION

Two data libraries have been extended and one new library has been added to the RSIC collection.

DLC-60B/MACKLIB-IV-82

The 171 neutron, 36 gamma-ray group nuclear response function library calculated with MACK-IV from ENDF/B-IV has been extended by Argonne National Laboratory. The library was generated using the same weight function as that used for DLC-41/VITAMIN-C and now contains values more consistent with the latter. In some cases, thermal group values were somewhat different from Maxwellian averages with the original MACKLIB-IV. Since VITAMIN-C and MACK-LIB-IV were developed primarily for fusion applications, such discrepancies were not easily noted. We are grateful to EG&G, Idaho for noting the problem. The updated version is denoted DLC-60B/MACKLIB-IV-82. Reference: ANL/FPP/ TM-106. IBM 370/3033.

DLC-74/PUDK-82

The library of measured results for delayed β and γ -ray spectra due to thermal neutron fission of ²³⁹Pu has been extended by Oak Ridge National Laboratory with the addition of simular data for ²⁴¹Pu. Results are included for irradiaton times of 1, 5, and 50 seconds. A retrieval code is supplied for editing the data in tabular form. References: ORNL/NUREG-34; NUREG/CR-0717 (ORNL/ NUREG-47). IBM 370/3033.

DLC-96/PEFPYD

A fission-product decay data library based on ENDF/B-IV was contributed by Los Alamos Scientific Laboratory, Los Alamos, New Mexico. The package contains two-point-per-decade aggregate fission-product decay-energy data and FITPULS, a code for obtaining analytic fits to the data. FIT- PULS is an interactive code which has not yet been made operable on the ORNL computers. It was developed on the LTSS used at LANL and LLNL. The following materials are included: ²³⁵U, ²³⁹Pu, ²³⁸U, ²³³U, ²⁴¹Pu, and ²³²Th. References: LA-8365-MS and LA-8277-MS. FORTRAN IV; CDC-6600.

VISITORS TO THE CENTER

During the month of May the following persons came for an orientation visit and/or to use EPIC facilities: Austin A. Odell, Lawrence Livermore National Laboratory; E. H. Brehm, BBC, Federal Republic of Germany; and J. G. Gonnord, CEA/CEN/Saclay, Gif-sur-Yvette, France.

COURSE OFFERING

Radiation Protection Short Course

A one week intensive course designed to provide an overview of the basic scientific and engineering principles underlying radiation protection is offered by Engineering Technology, Inc., Waco, Texas. The course is primarily descriptive, with a minimum of mathematics or theoretical rigor. It will be of value to practicing safety professionals, engineers, and technical managers who may be involved with radiation protection in an ancillary or management role, as well as to radiation protection technicians, radiographers, researchers, and others who work with ionizing radiations and radioactive materials. Topics to be discussed include: Atomic Structure and Radioactivity, Radioactive Decay, Production of X-Rays, Biological Effects of Radiation, Radiation Protection Standards, Legal Bases of Radiation Control, Radiation Units, Measurement of Radiation, Personnel Monitoring, Operations with Radioactive Materials, Contamination Control, and Disposal of Radioactive Wastes. Students should have a general understanding of Elementary Chemistry, Atomic Structure, and Physics, and should be able to perform simple algebraic calculations, including logarithmic and exponential functions.

Participants successfully completing the course will be awarded ABIH Maintenance Certification points and 3 Continuing Education Units.

Radiation Protection is offered on two occasions in 1982 during July 26-30 at Los Alamos, New Mexico, and October 25-29 at Anaheim, California. The \$650 fee for the five-day course includes course tuition, text and notes, meeting materials, and break refreshments. Registrations may be made by telephone and later confirmed by submitting the registration form and mailing it to: Short

Course Director, Engineering Technology, Inc., P.O. Box 9000, Waco, TX 76710. Telephone: (817) 772-0082.

CALENDAR

We call attention to the following courses and conferences.

July 1982

Power Plant Simulation (July 12-16) and Power Plant Simulation Workshop (July 19-21), short cources offered by the University of California, Los Angeles Extension. Either or both sessions may be attended. Contact Short Course Program Office, UCLA Extension, P.O. Box 24901, Los Angeles, CA 90024; phone (213) 825-1295.

Basic Use of Radionuclides in Research, Oak Ridge, Tennessee, July 19–30, 1982, sponsored by Oak Ridge Associated Universities, Inc. Contact: Registrar, Professional Training Programs, P.O. Box 117, Oak Ridge, TN 37830; phone (615) 576-3434.

IEEE Annual Conference on Nuclear and Space Radiation Effects, Las Vegas, Nevada, July 20-22, 1982, sponsored by: Institute of Electrical and Electronics Engineers, Inc.; NPSS Radiation Effects Committee; Defense Nuclear Agency; Jet Propulsion Laboratory; and Sandia National Laboratories. Contact: A. Ochoa, Jr., CRM Div. 2144, SNL, Albuquerque, NM 87185; phone (505) 844-6648, FTS 844-6648.

2nd Joint ASME/ANS Nuclear Conference, Portland, Oregon, July 25–28, 1982, sponsored by the American Society of Mechanical Engineers, New York. Contact: Larry Oyen, Sargent and Lundy, 55 E. Monroe, Chicago, IL 60603; phone (312) 269-2000.

MAY ACCESSION OF LITERATURE

The following literature has been selected for review, and that which is suitable will be placed in the RSIC Information Storage and Retrieval Information System (SARIS). This early announcement is made as a service to the shielding community. Copies of the literature are not distributed by RSIC. They may generally be obtained from the author or from a documentation center such as the National Technical Information Service (NTIS), Department of Commerce, Springfield, Virginia 22151.

RSIC maintains a microfiche file of the literature entered into SARIS, and duplicate copies of out-of-print reports may be available on request. Naturally, we cannot fill requests for literature which is copyrighted (such as books or journal articles) or whose distribution is restricted.

THIS LITERATURE IS NOT YET IN OUR SYSTEM. PLEASE ORDER FROM NTIS OR OTHER AVAILABLE SOURCE AS INDICATED.

REACTOR AND WEAPONS RADIATION SHIELDING LITERATURE

AECL-6433 Group Cross Sections in the Resolved Resonance Region Calculated for a CANDU-PHW Reactor Operating on Closed Thorium-Uranium and Thorium-Plutonium-Uranium Fuel Cycles.,.. Hamel, D.; Wilkin, G.B.,.. September 1979, . . Atomic Energy of Canada Ltd., Pinawa, Manitoba. Whiteshell Nuclear Research Establishment

ANL/ES-125 Estimated Dose to Man from Uranium Milling via the Terrestrial Food-Chain Pathway.,...Rayno, D.R.,...January 1982,...NTIS

BNL-50991, pp. 165-176; CONF-781174, pp. 165-176 Evaluation of High Plutonium Isotopes for the German Nuclear Data File KEDAK., ... Goel, B., ... May 1979, ... Brookhaven National Laboratory, Upton, NY

BNL-50991, pp. 275-286; CONF-781174, pp. 275-286 HEDL Evaluation of Higher Plutonium and Americium Cross Sections for ENDF/ B-V., . . Mann, F.M.; Schenter, R.E., . . May 1979, . . Brookhaven National Laboratory, Upton, NY, . . In: Proceedings of the Specialists' Meeting on Nuclear Data of Plutonium and Americium Isotopes for Reactor Applications, November 20-22, 1978. Chrien, R.E. (Ed.)

CONF-820321-3 Surveillance Dosimetry: Achievements and Disappointments., ... Wagschal, J.J.; Maerker, R.E.; Broadhead, B.L., ... 1982, ... NTIS, PC A02/MF A01, ... Portions of document are illegible. **CONF-820321-7** Uncertainties in the Estimation of Radiation-Damage Parameters.,...Stallmann, F.W.,... 1982,... NTIS, PC A02/MF A01

CONF-820321-9 Supplementary Neutron-Flux Calculations for the ORNL Pool Critical Assembly Pressure Vessel Facility., . . Maudlin, P.J.; Maerker, R.E., . . 1982, . . NTIS, PC A02/MF A01, . . Portions of document are illegible.

CONF-820321-10 Calculations of the Westinghouse Perturbation Experiment at the Poolside Facility., . . Maerker, R.E.; Williams, M.L., . . 1982, . . NTIS, PC A02/MF A01, . . Portions of document are illegible.

CONF-820321-15 Experience in Using the Covariances of Some ENDF/B-V Dosimetry Cross Sections: Proposed Improvements and Addition of Cross-Reaction Covariances., ... Fu, C.Y.; Hetrick, D.M., ... 1982, ... NTIS, PC A02/MF A01

CONF-820321-16 ANS Shielding Standards for Light-Water Reactors., . . Trubey, D.K., . . 1982, . . NTIS, PC A02/MF A01

CONF-820322-2 User Experience and Compatibility in Documentation Standards. Summary., . . Maskewitz, B.F., . . 1982, . . NTIS, PC A02/MF A01, . . Portions of document are illegible.

CONF-820335-2 Radiation Protection and Shielding Standards for the 1980s., . . Trubey, D. K., . . 1982, . . NTIS, PC A02/MF A01

DOE/EV/03382-20-Vol. I Radioactivity Studies. Progress Report. Vol. I., . . Cohen, N., . . September 1981, . . NTIS, PC A10/MF A01

DOE/EV/03382-20-Vol. II Radioactivity Studies. Progress Report. Vol. II., . . Cohen, N., . . September 1981, . . NTIS, PC A12/MF A01

EGG-9 Approach to Calculating Upper Bounds on Maximum Individual Doses from the Use of Contaminated Well Water Following a WIPP Repository Breach., ... Spiegler, P., ... September 1981, ... NTIS, PC A03/MF A01

EGG-PHYS-5615(Add. 1) Preliminary Calculations for Testing a Fission/Fusion Hybrid Blanket Module in a Fission Reactor. Addendum 1., .. Judd, J.L., .. December 1981, ... NTIS, PC A02/MF A01

EGG-PHYS-5668 Addendum to Integral Data-Testing Report for ENDF/B-V Dosimeter Cross Sections., . . Anderl, R.A.; Millsap, D.A.; Rogers, J.W.; Harker, Y.D., . . January 1982, . . NTIS, PC A04/MF A01

GA-A-16614 Radiation Hardening of Diagnostics for Fusion Reactors., . . Baur, J.F.; Engholm, B.A.; Hacker, M.P., . . December 1981, . . . NTIS

GJBX-30-82 Measured Time-Correlated Neutron-Induced Radiations in a Sandstone Formation. Final Report., . . Peters, C.; Karaoglan, E.; Ertel, J.; Brotzman, J.; Kennedy, C., Jr., . . July 1981, . . NTIS, PC A04/MF A01

HEDL-SA-2460-FP; CONF-811103-85 Proton Recoil Proportional Counter Measurements in FTR., . . Sloan, W.R.; McBreath, R.S.; Bennett, E.F.; Yule, T.J., . . 1981, . . NTIS, PC A02/MF A01

HEDL-SA-2467-S; CONF-811103-90 Absolute Fission Rates in the FFTF.,.. Gilliam, D. M.; Fuller, J.L.; Grundl, J.A.; Rawlins, J.A.; Daughtry, J.W., .. June 1981, .. NTIS, PC A02/ MF A01

HEDL-SA-2514; CONF-811145-8 Fundamental Radiation Effects Studies in the Fusion-Materials Program., . . Doran, D.G., . . October 1981, . . NTIS, PC A03/MF A01

HEDL-TME-81-45 Fusion Materials Irradiation Test Facility - Experimental Capabilities and Test Matrix., . . Opperman, E.K., . . January 1982, . . NTIS, PC A05/MF A01

EPRI-AP-2220 Low-Activation Structural Materials for Fusion Reactors - Extreme Purity Base Aluminum Alloys, Final Report., . . Kramer, R.A.; Franz, E.C.; Wagner, R.H., . . February 1982, . . Research Reports Center (RRC), Box 50490, Palo Alto, CA 94303

IFUSP-P-250 Modified Version of the Monte Carlo Computer Code for Calculating Neutron Detection Efficiencies., . . Nakayama, K.; Pessoa, E.F.; Douglas, R.A., . . December 1980, . . NTIS (U. S. Sales Only), PC A03/MF A01

INIS-SU-39 (In Russian) Problems on Dosimetry and Radiation Protection. No. 19., . . Ivanov, V.I. (Ed.), . . 1980, . . NTIS (U.S. Sales Only), PC A07/MF A01

JAERI-M-8769, pp. 2-16 (In Japanese) ENSDF (Evaluated Nuclear Structure Data File). Its Structure, Contents and Use in Applied Research., . . Martin, M.J.; Tamura, T.; Matumoto, Z.; Ohshima, M. (Eds.), . . March 1980, . . NTIS (U. S. Sales Only), PC A13/MF A01

JAERI-M-8769, pp. 195-209 (In Japanese) Level Density and Neutron Cross Sections., . . Yoshida, T.; Tamura, T.; Matumoto, Z.; Ohshima, M. (Eds.), . . March 1980, . . NTIS (U.S. Sales Only), PC A13/MF A01

Juel-1705 (In German); Thesis (In German) Neutronic Investigations on the Applica-

tion of Lithium Aluminates in the Tritium Breeding Blanket of Future Fusion Reactors. Attempts at Verifying Theoretical Predictions by Means of Special Measuring Methods... Mohsin, A.,.. February 1981,... NTIS (U.S. Sales Only), PC A06/MF A01

KFK-3033 (In German) DIAMANT2 - A Multigroup Neutron Transport Program for Triangular and Hexagonal Geometry., . . Kuefner, K.; Heger, R., . . September 1980, . . NTIS (U.S. Sales Only), PC A07/MF A01

LA-8113-MS Calculation of Cell Volumes and Surface Areas in MCNP., . . Hendricks, J.S., . . January 1980, . . NTIS, PC A03/MF A01

LA-8123-MS Impact of Vector Processors on Algorithms., . . Buxbee, B.L.; McKnight, A.L., . . November 1979, . . NTIS, PC A02/MF A01

LA-9193-T; Thesis Induced Compton Scattering Effects in Radiation Transport Approximations., . . Gibson, D.R., Jr., . . February 1982, . . NTIS

LA-9232-T; Thesis Two-Dimensional Cross-Section Sensitivity and Uncertainty Analysis for Fusion Reactor Blankets., . . Embrechts, M. J., . . February 1982, . . NTIS

NEANDC(E)-212U(V.5), pp. 25-26 Neutron Cross Sections for Actinides., . . Froehner, F. H.; Goel, B.; Qaim, S.M. (Eds.), . . June 1980, . . Fachinformationszentrum Energie, Physik, Mathematik. Karlsruhe, Germany, F.R.

NUREG/CR-2144; SAND-81-1142 A Compendium of Computer Codes for Light Water Reactor Analysis., . . Bieniarz, P.P.; Prassinos, P. G.; Engi, D., . . December 1981, . . GPO; NTIS

NUREG/CR-2626; ORNL/TM-8267 Ex-Core Neutron Detectors for Reactor Vessel Level Measurement., . . Anderson, J.L.; Anderson, R.L.; Miller, G.N., . . March 1982, . . GPO; NTIS

ORNL/TM-7717 Applications of Differential Sensitivity Theory for Extremum-Type Responses., . . Cacuci, D.G.; Maudlin, P.J.; Parks, C. V., . . May 1982, . . NTIS, PC A05/MF A01

ORNL/TM-8089 Thermal-Hydraulic Analysis of the Dual-Function Gamma Thermometer., . . Johnson, J.O.; Burns, T.J., . . December 1981, . . NTIS \$6.50

ORNL/TM-8109 Expected Very-near-Field Thermal Environments for Advanced Spent-Fuel and Defense High-Level Waste Packages., . . Rickertsen, L.D.; Misplon, M.A.; Claiborne, H.C., . . March 1982, . . NTIS, PC A04/MF A01 **ORNL/TM-8244** Toroidal Geometry Subroutines for MORSE-CG., . . Yamauchi, M.; Emmett, M.B.; Santoro, R.T.; Iida, H.; Seki, Y.; Narita, H., . . April 1982, . . NTIS, A04/MF A01

ORNL/TM-8258 Comparison of Measured and Calculated Neutron and Gamma Ray Energy Spectra Behind an In-Line Shielding Duct., . . Santoro, R.T.; Alsmiller, R.G., Jr.; Barnes, J.M.; Chapman, G.T.; Tang, J.S., . . May 1982, . . . NTIS, A03/ MF A01

RL-81-057; CONF-7806207-1 Neutron Transport from Targets to Moderators., . . Taylor, A.D., . . June 1981, . . NTIS (U.S. Sales Only), PC A03/MF A01

STI/PUB-521, pp. 437-449; CONF-790316, pp. 437-449 Is the Dose Equivalent Index a Practicable Concept for Use in Regulations... Holmberg, L.; Bengtsson, L.G.; Lindborg, L., .. 1979, .. IAEA, Vienna

STI/PUB-521, pp. 451-467; CONF-790316, pp. 451-467 Practical Implications of the Concept of Dose Equivalent Index., . . Kramer, R.; Drexler, G., . . 1979, . . IAEA, Vienna

UJV-4556 R,T,A (In Russian) Experimental and Computational Study of Neutron Field Characteristics in a Sodium Pipe Model. Part 2.,.. Burian, I.; Marek, M.; Osmera, B., et al., ... 1978, ... Nuclear Research Institute, Rez, Czechoslovakia

UJV-4930-R (In Czech) The Method of High-Order Local Polynomial Approximation for the Solution of the Multigroup Diffusion Equation in One- and Two Dimensional (r,z) Geometries., . Jakab, J., .. July 1979, .. Ustav Jaderneho Vyzkumu CSKAE, Rez, Czechoslovakia

WAPD-TM-1446 A Procedure for Obtaining Neutron Diffusion Coefficients from Neutron Transport Monte Carlo Calculations (AWBA Development Program)., . . Gast, R.C., . . August 1981, . . NTIS

XN-NF-75-27-NP, Suppl. 3 Exxon Nuclear Neutronics Design Methods for Pressurized Water Reactors, Supplement 3., . . No Author, . . February 1981, . . Exxon Nuclear Co., Inc., Richland, WA

XN-NF-80-56(NP) Generic Mechanical, Thermal Hydraulic and Neutronic Design for Exxon Nuclear Toprod Reload Fuel Assemblies for Pressurized Water Reactors... Brown, C.A.; Macduff, R.B.; Wimpy, P.D., .. December 1980, .. Exxon Nuclear Co., Inc., Richland, WA

Atomkernenergie, 35(3), 189-195 A Synthesis Method for Seed-Blanket Calculations., . Galperin, A.; Goldfeld, A.; Radkowsky, A.; Ronen, Y., . . 1980

Eroeterv Koezl., 17, 118-121 (In Hungarian) Shields Calculations., . . Pajer, I., . . 1979

Epites-Minoeseg, No. 5, 18-20 (In Hungarian) Testing of Radiation Shielding Concretes by Neutron Slowing-Down Method for the Paks Nuclear Power Plant., ... Toth, E., ... 1979

Health Phys., 42(3), 285-304 Organ Doses Due to External Gamma-Irradiation Arising from a Finite Plume in the Atmosphere., . . Gouvras, G.; Goddard, A.J.H., . . March 1982

Health Phys., 42(4), 489-495 Measurement of Neutrons Reflected from the Surfaces of a Calibration Room., . . Eisenhauer, C.M.; Schwartz, R.B.; Johnson, T., . . April 1982

Health Phys., 42(5), 565-584 Sensitivity Analysis of a Model for the Environmental Movement of Radionuclides., . . Helton, J.C.; Iman, R. L., . . May 1982

Health Phys., 42(5), 585-600 Calculation of the Concentration of Any Radionuclide Deposited on the Ground by Offsite Fallout from a Nuclear Detonation., . . Hicks, H.G., . . May 1982

Health Phys., 42(5), 717-723 Shielding Calculations Below 100 kVp.,.. Raghavendran, C. P.; Nagarajan, P.S.; Venkataraman, G., . . May 1982

J. Eng. Mater. Technol., 101(3), 182-190 Application of Corrosion Fatigue Crack Growth Rate Data to Integrity Analyses of Nuclear Reactor Vessels., . . Bamford, W.H., . . July 1979

Med. Phys., 9(1), 34-36 Unwanted Radiation Produced by Leakage Neutrons from Medical Electron Accelerators., . . Ing, H.; Shore, R.A., . . January 1982

Nucl. Technology, 57(3), 372-388 Neutron Emission Characteristics of Spent Boiling Water Reactor Fuel., . . Tsutomu, Y.; Tamura, T., . . June 1982

Nucl. Technology, 57(3), 436-441 Measurement of Neutron Fluence Above 0.1 with the Dosimeter $^{93}Nb(n,n')^{93m}Nb.$. Sakurai, K.,.. June 1982

Nucl. Sci. Eng., 81(1), 55-65 Sensitivity of Computed Uranium-238 Self-Shielding Factors to the Choice of the Unresolved Average Resonance Parameters., . . Munoz-Cobos, J.L.; Saussure, G. de; Perez, R.B., . . May 1982

Nucl. Sci. Eng., 81(1), 92-109 Numerical Analysis of the Boltzmann Equation Including Fokker-Planck Terms., ... Przybylski, K.; Ligou, J., ... May 1982 Nucl. Sci. Eng., 81(1), 110-118 Asymptotic Equivalence of Neutron Diffusion and Transport in Time-Independent Reactor Systems., . . Borysieqicz, M.; Mika, J.; Spiga, G., . . May 1982

Nucl. Sci. Eng., 81(2), 161-171 Measurements of Neutron and Gamma-Ray Streaming in a Cavity-Duct System and an Analysis by an Albedo Monte Carlo Method., . . Shin, K.; Murakami, R.; Taniuchi, H.; Hyodo, T.; Oka, Y., . . June 1982

Nucl. Sci. Eng., 81(2), 172-195 Effect of Fluorescence, Bremsstrahlung, and Annihilation Radiation on the Spectra and Energy Deposition of Gamma Rays in Bulk Media., ... Subbaiah, K.V.; Natarajan, A.; Gopinath, D.V.; Trubey, D.K., ... June 1982

Nucl. Sci. Eng., 81(2), 213-271 New Calculation of Prompt Fission Neutron Spectra and Average Prompt Neutron Multiplicities., . . Madland, D.G.; Nix, J.R., . . June 1982

Prog. Nucl. Energy, 5(1), 3-93 Decay Heat., . . Tobias, A., . . 1980

Strahlentherapie, 157(3), 187-190 (In German) Measurements and Calculation of the Air Activation Caused by 15 MeV Continuous Radiation Emitted by a Linear Accelerator.,.. Gremmel, H.; Ihnen, E.; Jensen, J.M., ... February 1981

Stud. Cercet. Fiz., 31(2), 129-135 (In Romanian) Energy Dependence of Gamma Photon Backscattering., . . Oncescu, M.C., . . 1979

Stud. Cercet. Fiz., 31(3), 331-343 (In Romanian) Sintering Diagram for Stoichiometric Uranium Dioxide... Georgeoni, P., .. 1979

Stud. Cercet. Fiz., 31(10), 1047-1060 (In Romanian) Backscattered Photon Angular Distribution for Oblique Incidence., . . Oncescu, M. C., . . 1979

Tr. Radiats. Gig., Leningr. Nauchno-Issled. Inst. Radiats. Gig., No. 7, 20-23 (In Russian) Dose Rate from the Square Volume Radiation Source., . . Karpov, V.I., . . 1978

Tr. Radiats. Gig., Leningr. Nauchno-Issled. Inst. Radiats. Gig., No. 7, 24-26 (In Russian) Gamma Radiation Inside Closed Volumes with Thin Irradiating Walls., . . Karpov, V.I., . . 1978

Thesis Constrained Finite Elements: A Coarse Mesh Technique for Solution of Neutron Transport Problems.,...Briggs, L.L.,...Northwestern University, Chicago, IL, ... 1978, ... University Microfilms Order No. 79-03,230

Thesis Application of Invariant Imbedding to Neutron Thermalization in Infinite Homogeneous Media., . . McCreless, T.G., . . Maryland University, College Park, MD, . . 1977, . . University Microfilms Order No. 78-11,944

BOOK REINFORCED CONCRETE DE-SIGNER'S HANDBOOK. 9th Edition., . . No Author, . . 1981, . . London; Cement and Concrete Association

BOOK FUSION RESEARCH., . . Dolan, T.J., . . 1982, . . Pergamon Press, New York

BOOK NEUTRON PHYSICS AND NUCLE-AR DATA IN SCIENCE AND TECHNOLOGY. VOLUME 1. NUCLEAR FISSION AND NEU-TRON-INDUCED FISSION CROSS-SECTIONS., . . Michaudon, A. (Ed.), . . 1981, . . Pergamon Press, Oxford, England

BOOK, pp. 157-221 Theoretical Methods for Calculating the Cross-Sections of Fissionable Nuclei. Chapter 5., .. Lynn, J.E., .. In: NEUTRON PHYSICS AND NUCLEAR DATA IN SCIENCE AND TECHNOLOGY. VOL. I. NUCLEAR FIS-SION AND NEUTRON-INDUCED FISSION CROSS-SECTIONS. Michaudon, A. (Ed.), ... 1981, .. Pergamon Press, Oxford, England, ... abg

COMPUTER CODES LITERATURE

- BNL 50991, 349-365 PLUTO Integral Experiments Performed at CEA to Improve the Pu Higher Isotope Nuclear Data: -PLUTO Programme-.,.. Chaudat, J.P.; Hammer, Ph.; Martin-Deidier, L., . . Centre d'Etudes Nucleaires de Cadarache, Saint-Paul-Lez-Durance, France, ... May 1979
- CEA-CONF-5278 (Pt.1) TRIPOLI-02
 The Three-Dimensional Monte-Carlo Code
 TRIPOLI-02., ... Baur, A.; Bourdet, L.; Dejonghe,
 G.; Gonnord, J.; Monnier, A.; Nimal, J.C.;
 Vergnaud, T., ... CEA Centre d'Etudes Nucleaires
 de Saclay, 91-Gif-sur-Yvette, France, ... April 1980
- CEA-CONF-5278 (Pt.2) TRIPOLI-02
 The Three-Dimensional Monte-Carlo Code
 TRIPOLI-02., ... Baur, A.; Bourdet, L.; Dejonghe,
 G.; Gonnord, J.; Monnier, A.; Nimal, J.C.;
 Vergnaud, T., ... CEA Centre d'Etudes Nucleaires
 de Saclay, 91-Gif-sur-Yvette, France, ... April 1980
- CEA-CONF-5278 (Pt.3) TRIPOLI-02
 The Three-Dimensional Monte-Carlo Code
 TRIPOLI-02., ... Baur, A.; Bourdet, L.; Dejonghe,
 G.; Gonnord, J.; Monnier, A.; Nimal, J.C.;
 Vergnaud, T., ... CEA Centre d'Etudes Nucleaires
 de Saclay, 91-Gif-sur-Yvette, France, ... April 1980

- CONF-791103-91 GAM-THERMOS; SCALE
 SCALE System Cross Section Validation with
 Shipping-Cask Critical Experiments., . . Westfall,
 R.M.; Knight, J.R., . . Oak Ridge National
 Laboratory, TN, . . November 1979
- DNA 5159F-1DELFIC DELFIC: Department of Defense Fallout Prediction System. Volume 1: Fundamentals., ... Norment, H.G., ... Atmospheric Science Associates, Bedford, Mass. for Defense Nuclear Agency, Washington, D.C., ... December 1979
- DNA 5159F-2DELFIC DELFIC: Department of Defense Fallout Prediction System. Volume II: User's Manual., .. Norment, H.G., .. Atmospheric Science Associates, Bedford, Mass. for Defense Nuclear Agency, Washington, D.C., .. December 1979
- EIR-365 SURCU; SURCU-OH The DPN –Surface Flux– Integral Transport Method with General P_n Polynomial Approximation.,.. Stepanek, J.,.. Eidg. Institut fur Reaktorforschung, Wurenlingen, Switzerland, ... March 1979
- EPRI NP-926 OZMA OZMA - A Code to Calculate Resonance Reaction Rates in Reactor Lattices Using Resonance Profile Tabulations., . . Technion - Israel Institute of Technology, Israel, . . February 1981
- EPRI NP-1635 SAM-CE Improvement of the SAM-CE Criticality Capability and Analysis of Thermal Reactor Benchmarks.,... Mathematical Applications Group Inc., Elmsford, NY for Electric Power Research Institute, Palo Alto, CA,... November 1980

Nippon Genshiryoku Gakkai-Shi., 21(5), 377-396 (In Japanese)

... POPOP4; TWOWAY; JENDL; DUCAL; GROGI; CASTHY; HELGA; TNG Photon Production Nuclear Data for Reactor Design., . . Japanese Nuclear Data Committee, Tokai, Ibaraki, Japan, . . May 1979

- Nucl. Technol. 47(1), 200-207 DOT IV Radiation Transport in Earth for Neutron and Gamma-Ray Point Sources Above an Air-Ground Interface., . . Lillie, R.A.; Santoro, R.T., . . Oak Ridge National Laboratory, TN, . . January 1980
- ORNL/TM-8244 MORSE-CG Toroidal Geometry Subroutines for MORSE-CG., . . Yamauchi, M.; Emmett, M.B.; Santoro, R.T.; Iida, H.; Seki, Y.; Narita, H., . . Oak Ridge National Laboratory, TN, . . April 1982
- PTB-FMRB-71 (In German) NUKLID NUKLID - A Program for the Automatic Identification of Lines in Ge(Li) Spectra., . . Guenther, E., . . Physikalisch-Technische

Bundesanstalt, Braunschweig, West Germany, . . March 1978, . . AVAIL: NTIS (U.S. Sales Only)

- STUDSVIK-RF-77-2268 (In Swedish) ... BEGAFIP Updating of BEGAFIP: Computer Program for Calculation of the Activity of Fission Products in Spent Nuclear Fuel., .. Ekberg, K.; Olsson, G., .. Aktiebolaget Atomenergi, Studsvik, Sweden, ... March 1978, ... AVAIL: NTIS (U.S. Sales Only)
- Thesis, Univ. Microfilms Order No. 80-20,229 VIM; EPRI-LEOPARD; BGIM Efficient Method for the Solution of the Energy Dependent Integral Boltzmann Transport Equation in the Resolved Resonance Energy Region., ... Schwenk, G.A., ... Virginia Polytechnic Inst., Blacksburg, VA, ... 1980