

No. 200

August 1981

Information conserves other resources through better decisions...John Richardson

ANS/RP&S HONORS CAPO AND RHOADES

At its June 1981 meeting in Miami the American Nuclear Society Radiation Protection and Shielding Division presented awards for outstanding technical achievement to Mary Ann Capo of Offshore Power Systems and Wayne A. Rhoades of Oak Ridge National Laboratory.

The citations were presented by Gerald P. Lahti, Chairman of the ANS Radiation Protection and Shielding Division, and read as follows:

On behalf of the American Nuclear Society, the Radiation Protection and Shielding Division is pleased to present to

Mary Ann Capo

its Award for Outstanding Service to the Society, to the Division, and to the radiation transport and shielding community worldwide.

The award is made in recognition of her achievements in radiation protection and radiation shielding, particularly in regards to her achievements in reducing Monte Carlo transport and point kernel methodologies to routine practice for a wide spectrum of radiation shielding problems. Since her initial involvement in the development and application of numerous transport methods and data bases useful in radiation transport analyses for the aircraft nuclear propulsion, she has been instrumental in the development of radiation shielding analysis methodologies for nuclear rocket propulsion applications, floating nuclear power plants, and numerous other nuclear energy applications. Her ability to recognize the practicality of an analysis approach and the interpretation of shielding experiment to provide qualified design methods has been an outstanding contribution to the radiation shielding community.

Ms. Capo's involvement in the engineering applications of computer programs and nuclear data was developed with an insight of user requirements that proved to withstand the passage of time. Nuclear data and methods developed under her cognizance have been passed to the next generation of radiation shielding analysts.

Highlights of her work over the years in advancing the technology of radiation protection and shielding are as follows:

Development and validation of engineering applications of Monte Carlo radiation transport methods for radiation shielding and environment studies;

Development of simplified data bases, e.g., buildup factors, gamma ray cross sections, applicable to engineering

applications using computer methods;

Sound assessment of radiation transport methods for the diverse set of radiation shielding problems encountered in nuclear powered aircraft, nuclear space propulsion, floating central station nuclear power;

Pioneering the practical application of discrete ordinates transport methods for radiation streaming in a PWR cavity with direct involvement in the validation of the methodology using experimental data from operating commercial nuclear power plants;

Development of the application and validation of assymetric (highly biased) angular quadrature methods in discrete ordinate transport to complex radiation transport analyses for nuclear rocket propulsion.

Development of the source terms for consequence analysis for the first Class 9 accident liquid pathway transport study.

Ms. Capo's work in performance and management of the development of applications for radiation transport methods and data bases for radiation shielding design analysis has always involved new and complex nuclear energy applications. Her contributions have provided a means to use computer methods to routinely solve complex radiation transport problems where weight, space, and/or cost are primary design objectives.

On behalf of the American Nuclear Society, the Radiation Protection and Shielding Division is pleased to present to

Wayne A. Rhoades

its award for Technical Achievement by virtue of his leadership in radiation transport methods development.

His leadership in the development of two-dimensional discrete ordinates methods for solving radiation-shielding problems has been outstanding. DOT-IV, the two-dimensional discrete ordinates radiation transport code which was developed under Rhoades' leadership, is an exceptionally efficient multi-dimensional code as evidenced by the demand for DOT-IV in the shielding community throughout the world.

He has extended existing techniques and developed new techniques which have proven to be powerful tools used in the acceleration of the convergence of the iteration process in the discrete ordinates method. He has developed modified flux model equations and techniques for effective removal of negative source terms which improve the accuracy and usability of the discrete ordinates method. He has led the investigation, development, and application of very efficient input/output methods used in the DOT-IV code. His continuing quest for the most efficient style of programming consistent with the traditional versatility of the DOT program has resulted in the most efficient code in that series.

In addition to his direct contributions, he continues to have an important influence on the work of others through his consultations on methods development, particular applications of the DOT-IV program, and the efficient, innovative use of current computing machinery.

ANS Best Paper Award

Wilber Bunch of HEDL, Chairman of the ANS Radiation Protection and Shielding Division Honors and Awards Committee, has announced the winner of the Best Paper Award following presentation at the June 1981 meeting at Miami, Florida. The winning paper, prepared by M. E. Sawan, G. A. Moses, and G. L. Kulcinski of the University of Wisconsin, Madison, was titled "Time-Dependent Radiation Damage to the First Wall of a Heavy Ion Beam Fusion Reactor with Porous Tube Protection." Congratulations to the winning authors who share a cash prize.

CHANGES IN THE COMPUTER CODE COLLECTION

The following changes were made to the codes collection during July.

CCC-400/DOSFACTER II

DOSFACTER II, based on DOSFACTER (CCC-337), a calculation of dose-rate conversion factors for exposure to photons and electrons, was contributed by Oak Ridge National Laboratory. In addition to the DOSFACTER code package (CCC-337) capabilities, DOSFACTER II includes (a) calculation of electron dose-rate factors for radiosensitive portions of the skin; (b) incorporation of improved estimates of organ

dose-rate factors for photons; and (c) calculation of dose-rate factors for additional radionuclides and incorporation of updated radioactive decay data for all radionuclides. External dose-rate factors for photons and electrons have been calculated for approximately 500 radionuclides, and those results are packaged separately as DLC-79/DOSDAT II-81. References: Trans. Am. Nucl. Soc., 34, 84 (1980) and ORNL/NUREG-79 (NUREG/CR-1918). FORTRAN IV; IBM 360/370.

SCA-2/KENO-IV

The authors of the KENO-IV Monte Carlo criticality code have notified RSIC of changes made to SCA-2/KENO-IV. SCA-2/KENO-IV has not been distributed since October 16, 1980 when it was superceded by the KENO-IV which is included in the SCA-0/SCALE code package. This announcement is to inform persons who are using the SCA-2 version of KENO-IV that they may request from RSIC details of these recent changes.

CHANGES IN THE DATA LIBRARY COLLECTION

The following changes were made in the data collection in July.

DLC-84/MENSLIB

The 60-neutron-group cross-section library was contributed by Los Alamos National Laboratory and Texas A&M University. The data are for H, B, C, N, O, Si, Fe, and W in P₅ expansion in the "DTF-IV" format, i.e., the user must multiply the cross sections by (2H1) if used in codes that accept "ANISN" format. Data below 20 MeV were processed from ENDF/B-IV using MINX and at higher energies using DANA5. The library was used in the design of a shield-collimator unit for fast neutron radiotherapy. Reference: LA-7159-T (Thesis).

DLC-86/FLUNG

The 35-neutron, 21-gamma-ray group library was derived from DLC-41/VITAMIN-C and utilized in a variety of fusion neutronics applications in the Engineering Physics Division at ORNL. Data are provided for the 61 materials available from VITAMIN-C (ENDF/B-IV). A retrieval code for BCD-to-binary conversion is provided. IBM-370/3033. Reference: ORNL/TM-7828.

DLC-87/HILO

The 66-neutron, '21-gamma-ray-group cross-section library for neutron transport up to 400MeV was contributed by Engineering Physics Division, Oak Ridge National Laboratory. Below 15 MeV the library consists of data derived from DLC-41/VITAMIN-C. Above that energy, differential elastic cross-section data from optical model calculations and nonelastic data from intranuclear-cascade-evaporation model are used. The library is P_3 above 15 MeV, P_3 below, in ANISN format, with a retrieval code for BCD to binary conversion. Data are provided for H, ¹⁰B, ¹¹B, C, N, O, Na, Mg, Al, Si, S, K, Ca, Cr, Fe, Ni, W, and Pb. IBM-360/370. Reference: ORNL/TM-7818.

WHAT HATH ICRU WROUGHT?

We have had a copy of ICRU Report 33, "Radiation Quantities and Units" for several months, but we did not notice what the ICRU had done with the word *flux* until we read the book review of F. H. Attix in the July issue of *Health Physics*. He calls our attention to the new ICRU definitions of *particle number*, N (the number of particles emitted, transferred, or received) and *particle flux*, dN/dt (the quotient of the increment of particle number, dN, in time interval dt). This *flux* is quite different from the similar term *flux* density, or the ICRU-preferred term *fluence rate*. We believe that the new definition will lead to considerable confusion in the literature since, quoting the ANS-9/ANSI N1.1–1976 glossary, the *particle flux density* "is also commonly, but incorrectly, called *flux*." We refer the reader to the Attix review for additional commentary on the new ICRU pronouncements.

DEATH OF ED CASHWELL NOTED

It is with profound regret that we note the death of Edmond D. Cashwell on July 11, 1981 at age 61. His scientific contributions, however, will continue to live on indefinitely.

After receiving his Ph.D. in mathematics from the University of Wisconsin in 1949, Cashwell came to Los Alamos in 1951. He followed the early pioneers at Los Alamos, von Neumann, Ulam, Metropolis, and Richtmyer, in being responsible for development of the theoretical Monte Carlo method for computing radiation transport which further enhanced Los Alamos as a leading center for such work.

Ed was singularly outstanding for his publications which have provided much of his knowledge to the rest of us who are today solving problems using the Monte Carlo method. His publications include:

- 1. E. D. Cashwell and C. J. Everett, "A Practical Manual on the Monte Carlo Method for Random Walk Problems," Pergamon Press, Inc., New York (1959). [Also LA-2120 (1957)].
- 2. E. D. Cashwell and C. J. Everett, "A Monte Carlo Sampler," LA-5061-MS (1972).
- 3. E. D. Cashwell and C. J. Everett, "A Second Monte Carlo Sampler," LA-5723-MS (1974).
- 4. L. L. Carter and E. D. Cashwell, "Particle-Transport Simulation with the Monte Carlo Method," ERDA Critical Review Series, TID-26607 (1975).
- 5. C. J. Everett and E. D. Cashwell, "A Relativity Primer for Particle Transport, A LASL Monograph," LA-7792-MS (1979).
- 6. LASL Group X-6, "MCNP-A General Monte Carlo Code for Neutron and Photon Transport," LA-7396-M, Revised (1979).

Cashwell was Group Leader of the Los Alamos Monte Carlo Group from 1971 through 1974. In April 1980, he was given a Los Alamos Distinguished Performance Award for his outstanding contributions in the field of Monte Carlo. In February 1981, he became a Fellow at the Laboratory.

We will long remember Ed, not only for his technical expertise, but for his integrity and graciousness.

STANDARD ON LWR AREA MONITORING SYSTEMS PUBLISHED

A new standard, written by Working Group ANS-HPSSC-6.8 led by Delbert J. Schuh, II, Fabricated Systems, Inc. and titled "Location and Design Criteria for Area Radiation Monitoring Systems for Light Water Reactors." This has been a joint project of the American Nuclear Society and Health Physics Society standards committees.

The scope reads as follows:

This standard provides criteria for the establishment of locations for fixed continuous area gamma radiation monitoring instrumentation for use in light water reactors. It also provides criteria for design features and ranges of measurement for such instrumentation which are consistent with current and anticipated plant designs.

This standard applies only to area gamma radiation monitoring during routine operations, including anticipated operational occurrences. The following are not within the scope of this standard:

- (1) Monitoring during accident and post accident conditions;
- (2) The detection of ambient levels of other than gamma radiation;
- (3) Control and operator actions, if any, resulting from the area radiation measurement;
- (4) Portable health physics survey instruments;
- (5) Personnel monitoring devices;
- (6) Instrumentation used solely for the monitoring of various plant processes; and
- (7) The determination of alarm setpoints.

The standard, designated ANSI/ANS-HPSSC-6.8.1-1981, is available from the American Nuclear Society, 555 N. Kensington Ave., LaGrange Park, IL 60525 for \$20 per copy.

The working group is now proceeding to develop a similar standard on process monitors.

BNL Seminar/Workshop Postponed

The BNL seminar/workshop on *Thermal Reactor Benchmark Calculations, Techniques, Results, and* Applications announced in July newsletter has been postponed to May 17-18, 1982 from October 19-20, 1981.

THOMAS A. JAEGER PRIZE ANNOUNCED

The Commission of the European Communities and the International Association for Structural Mechanics in Reactor Technology e.V., Berlin (Germany), announce the establishment of a prize to encourage and stimulate research in structural mechanics applied to reactor technology. The underlying purpose is to promote the technological bases for safe and economic operation of nuclear fission reactors and bring nearer the advent of nuclear fusion energy.

This prize is created in honour of Professor Thomas A. Jaeger and will bear his name. He was the founding Director of the International Association for Structural Mechanics in Reactor Technology e.V. The prize shall be awarded every two years on the occasion of the SMiRT Conferences, the first such award being made during the sixth Conference, which will be held in Paris in 1981. The authors of communications at this conference are qualified to enter the contest for the Thomas A. Jaeger Prize.

This SMiRT-6 Conference is dedicated to the memory of Professor Th. A. Jaeger, who died on August 21, 1980. Thomas A. Jaeger founded the SMiRT Conferences and ensured the success of all of them up to the SMiRT-5 Conference in Berlin. He convinced everyone concerned of the importance of the cross-fertilization of ideas in structural mechanics, for the safety, reliability and good performance of nuclear reactors, and the organizers are doing their utmost to organise SMiRT-6 the way that the founder would have wanted it. By the value of the exchanges that it will initiate, this conference is their best tribute to the memory of this exceptional man, a greatly missed friend.

Although primarily a structural engineer, Tom had a keen interest in shielding. He was one of the leaders and an editor of the largest work ever published on shielding: the *Engineering Compendium on Radiation Shielding*, published by Springer-Verlag in three volumes under sponsorship of the International Atomic Energy Agency (IAEA). The board of editors was comprised of R. G. Jaeger (editor-in-chief), E. P. Blizard, A. B. Chilton, M. Grotenhuis, A. Hönig, T. A. Jaeger, and H. H. Eisenlohr.

As co-editor of *Nuclear Engineering and Design*, an international journal devoted to the thermal, mechanical and structural problems of nuclear energy, Tom worked diligently in the early years to promote that journal as a vehicle for shielding literature, but a few years ago the decision was made for the journal to concentrate on other topics.

PERSONAL ITEMS

Nancy C. Smith has joined the Engineering Physics Information Center (EPIC) staff to head the preparation of publications, and this newsletter is one of her first products. She replaces Pam Young who has left ORNL employment. Nancy transferred from the Technical Publications department of the ORNL Information Division where she has been involved in publications work for a number of years.

Professional Divisions Name New Officers

The ANS professional divisions below have announced the names of their new officers for 1981-82. In the following list, the names of those officers designated chair-elect last year are given in parentheses.

Radiation Protection and Shielding—(Chair, Michael J. Kolar); vice chair/chair-elect, David E. Bartine; secretary, E. Thomas Boulette; treasurer, Joseph M. Cardito; executive committee I, Robert T. Santoro; executive committee II, Leland L. Carter; executive committee III, Victor J. Orphan.

Reactor Physics (Chair, Burt A. Zolotar); vice chair/chair-elect, Michael J. Lineberry; secretary, Rudolph Sher; treasurer, Surendra N. Purohit; executive committee, Roger Rydin, Odelli Ozer, and David C. Wade.

Mathematics and Computation—(Chair, John E. Meyer); vice chair/chair-elect, Richard J. Pryor; secretary, Lawrence L. Barinka; treasurer, I. K. Abu-Shumays; executive committee, Patrick C. Bailey, George H. Miley, and R. Douglas O'Dell.

Isotopes and Radiation—(Chair, Donald E. Wood); vice chair/chair-elect, Michael C. Stauber; secretary, W. A. Jester; treasurer, Eric T. Clark; executive committee, Richard C. Ragaini, Charles C. Thomas, Jr., and William E. Kreger.

VISITORS TO EPIC

The following persons came for an orientation visit and/or to use EPIC facilities during the month of July: Shiaw-Der Su, General Atomic Co., (currently on contract at TVA, Knoxville); Hans-Peter Weise, Bundesanstalt für Materialprüfung (BAM), Berlin, Germany; Kenneth Kite, Nuclear Assurance Corporation, Atlanta, Georgia; M. Moghari, Control Data Corporation, Rockville, Maryland; Lowell K. Reed, Boeing, Oak Ridge, Tenn.; and Dermott E. Cullen, IAEA, Vienna, Austria.

JULY ACCESSION OF LITERATURE

The following literature cited has been ordered for review, and that selected as suitable will be placed in the RSIC Information Storage and Retrieval Information System (SARIS). This early announcement is made as a service to the shielding community. Copies of the literature are not distributed by RSIC. They may generally be obtained from the author or from a documentation center such as the National Technical Information Service (NTIS), Department of Commerce, Springfield, Virginia 22151.

RSIC maintains a microfiche file of the literature entered into SARIS, and duplicate copies of out-of-print reports may be available on request. Naturally, we cannot fill requests for literature which is copyrighted (such as books or journal articles) or whose distribution is restricted.

THIS LITERATURE IS ON ORDER. IT IS NOT IN OUR SYSTEM. PLEASE ORDER FROM NTIS OR OTHER AVAILABLE SOURCE AS INDICATED.

REACTOR AND WEAPONS RADIATION SHIELDING LITERATURE

AERE-R-8961

A Study of the Temperature Dependence of the Neutron Transmission of Uranium Dioxide., Haste, T.J.; Sowerby, M.G., January 1978, HMSO

ANL/FPP/TM-127, pp.54-59

Shielding and Maintainability in an Experimental Tokamak., Abdou, M.A.; Guller, G.; Hager, E.R.; Vogelsang, W.F., November 1979, NTIS

BNL-NCS-29035; CONF-800979-16

New Aspects in the Evaluation of Thermal Neutron Cross Sections, Mughabghab, S.F., 1980, NTIS, PC A03/MF A01

CEA-R-4916 (In French)

Finite Element Methods for Static Transport Calculations., Gerin-Roze, J., August 1978, CEA Centre d'Etudes de Limeil, 94 - Villeneuve-Saint-George (France)

CEA-CONF-4917 (In French); CONF-7906188, 2 pages

Use of Cross Section Library in the Field of Safety., Duchemin, B.; Nimal, J.C., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CEA-CONF-4918 (In French); CONF-7906188-2(Summ.)

Use of Fission Product Library for After-Heat and Activity Calculations., Duchemin, B.; Nimal, B.; Nimal, J.C.; Veillaut, J.P., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CEA-CONF-4986; NBS Spec. Pub. 594, pp.194-198; CONF-791058-77, pp.194-198

Nuclear Data for Shielding Calculations; Na Cross-Section Adjustment Using Propagation Experiments., Estiot, J.C.; Salvatores, M.; Trapp, J.P.; Carli, A. de; Rado, V., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CEA-CONF-4987; NBS Spec. Pub. 594, pp.190-193; CONF-791058-78, pp.190-193

Basic Nuclear Data and Reactor Shielding Design Formulaire PROPANE Do., Estiot, J.C.; Salvatores, M.; Trapp, J.P., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CINDA 81 (1977-1981)

An Index to the Literature on Microscopic Neutron Data., IAEA, May 1981, International Atomic Energy Agency, Vienna

CONF-790222-1

Recent Trends in Radiation Shielding: An RSIC Perspective., Trubey, D.K.; Roussin, R.W.; Maskewitz, B.F., February 1979, NTIS, PC A02/MF A01

CONF-810406-7

Methodologies for Calculating Measurement Uncertainties., Lassahn, G.D., 1981, NTIS, PC A02/MF A01

CONF-810462-2

Validation of Annual Average Air Concentration Predictions from the AIRDOS-EPA Computer Code., Miller, C.W.; Fields, D.E.; Cotter, S.J., 1981, NTIS, PC A02/MF A01

DOE/EML-391

Iterative Nonlinear Unfolding Code: TWOGO., Hajnal, F., March 1981, NTIS, PC A05 MF A01

EPRI-NP-1862

Occupational Radiation-Exposure Reduction Technology Planning Study. Final Report (Technical Planning Study TPS 79-761)., Warman, E.A.; McMellon, R.M.; Cardito, J.M., May 1981, Research Reports Center(RRC), Box 50490, Palo Alto, CA 94303, Tel. (415)965-4081

IAEA-207, pp.1-11; CONF-761081, pp.1-11 Summary Report., Butler, J., In: Differential and Integral Nuclear Data Requirements for Shielding Calculations., 1978, NTIS

IAEA-207, pp.17-21; CONF-761081, pp.17-21 Target Accuracies., Herrnberger, V., 1978. NTIS

IAEA-207, pp.23-26; CONF-761081, pp.23-26 Conclusions from the Sensitivity Benchmark Results of a PWR Shield., Hehn. G., 1978, NTIS

IAEA-207, pp.27-30; CONF-761081, pp.27-30 Conclusions Concerning the "Benchmark Problem for Fast 'Reactor Shielding"., Khairallah, A., 1978, NTIS

IAEA-207, pp.31-33; CONF-761081, pp.31-33 Methodology of Sensitivity Analysis., Beynon, T.D., 1978, NTIS

IAEA-207, pp.35-38; CONF-761081, pp.35-38 Review of the Experimental Benchmark Programme., Nicks, R., 1978, NTIS IAEA-207, pp.41-44; CONF-761081, pp.41-44 Design Sensitivity and Uncertainty Analysis of Fast Breeder Reactor Shields., Salvatores, M., 1978, NTIS

IAEA-207, pp.45-48; CONF-761081, pp.45-48 Summary of the Conclusions of the Subgroup on "Sensitivity Studies and Data Requirements for Thermal Reactor Shielding"., Devillers, C., 1978, NTIS

IAEA-207, p.55; CONF-761081, p.55 Summary Remarks from the IAEA Nuclear Data Section., Schmidt, J., 1978, NTIS

IAEA-207, pp.59-71; CONF-761081, pp.59-71 Target Accuracies Required in Radiation Shielding., Herrnberger, V.; Hehn, G.; Nicks, R., 1978, NTIS

IAEA-207, pp.73-88; CONF-761081, pp.73-88 Target Accuracies and Sensitivity Studies in the Assessment of Data Requirements for Practical Shield Design., Avery, A.F., 1978, NTIS

IAEA-207, pp.91-124; CONF-761081, pp.91-124 Benchmark Calculations for 1300 MWe PWR Type Reactors. Sensitivity of the Shielding Calculations., Brandicourt, G.; Devillers, C., 1978, NTIS

IAEA-207, pp.147-154; CONF-761081, pp.147-154 Two NEA Sensitivity, 1-D Benchmark Calculations. Part II: Sensitivity of the Secondary Sodium Activation in the FBR Configuration., Canali, U.; Gonano, G.; Nicks, R., 1978, NTIS

IAEA-207, pp.155-174; CONF-761081, pp.155-174 Contribution to the Study of the Reactor Shielding Benchmark No.2 (PWR) Proposed by the NEA., Minsart, G.; Bosstraeten, C. van, 1978, NTIS

IAEA-207, pp.175-195; CONF-761081, pp.175-195 Contribution to the Exercise on Sensitivities for the NEA Theoretical PWR Benchmark., Lympany, S.D.; McCracken, A.K.; Packwood, A., 1978, NTIS

IAEA-207, pp.197-220; CONF-761081, pp.197-220 Neutron Induced Gamma-Ray Sources in Shielding Benchmark Calculations., Miyasaka, S.; Koyoma, K.; Minami, K.; Tsuzuki, S., 1978, NTIS

IAEA-207, pp.221-241 (In French); CONF-761081, pp.221-241 (In French)

Benchmark Problems for the Shieldings of a Fast Neutron Reactors., Estiot, J.C.; Trapp, J.P., 1978, NTIS IAEA-207, pp.245-259 (In French); CONF-761081, pp.245-259 (In French)

Sensibility Studies of the Equivalent Thermal Neutron Flux on the Heat Exchanger of a Sodium Cooled Fast Reactor. Pt.1., Boioli, A.; Fiorini, L.; Moreau, J.; Suntinger, G., 1978, NTIS

IAEA-207, pp.261-273; CONF-761081, pp.261-273 Recent Progress at ORNL in Determining Nuclear Data Requirements for Fast Reactor Shield Design Using Advanced Sensitivity Techniques., Oblow, E.M.; Weisbin, C.R., 1978, NTIS

IAEA-207, pp.275-283; CONF-761081, pp.275-283 Investigation of Cross Section Requirements for Shielding of Power Reactors., Hehn, G.; Rataj, J., 1978, NTIS

IAEA-207, pp.285-322; CONF-761081, pp.285-322 Contribution to the Exercise on Sensitivity Studies for the NEA Theoretical Fast Reactor Benchmark., Lympany, S.D.; McCracken, A.K.; Packwood, A., 1978, NTIS

IAEA-207, pp.345-351; CONF-761081, pp.345-351 Sensitivity Studies in 3D Shielding Configurations., Rief, H., 1978, NTIS

IAEA-207, pp.353-357; CONF-761081, pp.353-357 Some Requirements as Regards the Accuracy of Constants for Fast Power Reactor Shielding Calculations., Bakumenko, O.D.; Kulakovský, M.Ya.; Savitsky, V.I., 1978, NTIS

IAEA-207, pp.359-365; CONF-761081, pp.359-365 The Track Rotation Estimator and Its Application to Shielding Benchmark Experiments., Dubi. A.; Horowitz, Y.S., 1978, NTIS

IAEA-207, pp.367-376; CONF-761081, pp.367-376 Integral Tests of Coupled Multigroup Neutron and Gamma Cross Sections with Fission and Fusion Sources., Schriewer, J.; Hehn, G.; Mattes, M.; Pfister, G.; Keinert, J., 1978, NTIS

IAEA-207, pp.421-428; CONF-761081, pp.421-428 International Nuclear Data Activities Related to Shielding Calculations., Lessler, R.M., 1978, NTIS

IAEA-207, pp.431-463 (In French); CONF-761081, pp.431-463 (In French)

Results of Neutron-Propagation Experiments Performed in Iron-Sodium Mixtures Using the Source Reactor HARMONIE and TAPIRO., Boutcau, F.; Calamand, D.; Oceraies, Y.; Vienot, R.; Antonini, D.; Bozzi, L.; Martini, M.; Moioli, P., 1978, NTIS Analysis of Neutron Propagation Experiments in Iron-Sodium Mixtures with the PROPANE Calculational Scheme for Fast Reactors., Estiot, J.C.; Trapp, J.P.; Salvatores, M.; Palmiotti, G., 1978, NTIS

IAEA-207, pp.505-530; CONF-761081, pp.505-530 Two-Dimensional Shielding Benchmarks for Iron at YAYOI., Oka, Y.; An, S.; Kasai, S.; Yoshii, R.; Hashikura, H.; Akiyama, M.; Miyasaka, S.; Koyama, K.; Hyodo, T., 1978, NTIS

IAEA-207, pp.531-542; CONF-761081, pp.531-542 100-Group, 2-D Calculation of the Winfrith Iron Benchmark Experiment., Burn, K.; Canali, U.; Nicks, R., 1978, NTIS

IAEA-207, pp.563-572; CONF-761081, pp.563-572 Evaluation of the Effect of Errors in Group Constants on Calculations of Neutron Fields in Shielding., Germogenova, T.A.; Suvorov, A.P.; Utkin, V.A., 1978, NTIS

JAERI 1268; NEANDC(J)-68/U; INDC(JAP)-55/L Neutron Cross Sections of 28 Fission Product Nuclides Adopted in JENDL-1., Kikuchi, Y.; Nakagawa, T.; Matsunobu, H.; Kawai, M.; Igarasi, S.; Iijima, S., February 1981, Division of Technical Information, Japan Atomic Energy Research Institute, Tokai-mura, Naka-gun, Ibaraki-ken, Japan

JAERI-M-8556 (In Japanese)

Computer Programme for Perturbation Calculations by Correlated Sampling Monte Carlo Method., Nakagawa, M.; Asaoka, T., November 1979, NTIS (U.S. Sales Only), PC A03/MF A01

LA-8830

Nucleonic Analysis of the ETF Neutral-Beam-Injector-Duct and Vacuum-Pumping-Duct Shields., Urban, W.T.; Seed, T.J.; Dudziak, D.J., May 1981, NTIS, PC A05/MF A01

LUND6-NFFR-3030/1-37/1979

NENINA - A Computer Code for the Calculation of Energy Distributions of Neutrons Incident on a Circular Disc., Magnusson, G.; Andersson, P., April 1979, Fysiska Institutionen, Lund Univ. (Sweden)

M-100

Multigamma-Ray Calibration Sources., Meyer, R.A., December 1978, NTIS, PC A04/MF A01

NUREG/CR-1769; PNL-3585

Neutron Dosimetry at Commercial Nuclear Plants. Final Report of Subtask A: Reactor Containment Measurements., Endres, G.W.R.; Aldrich, J.M.; Brackenbush, L.W.; Faust, L.G.; Griffith, R.V.; Hankins, D.E., May 1981, NTIS; GPO \$5.50

NUREG/CR-2011; PNL-3767

MILDOS - A Computer Program for Calculating Environmental Radiation Doses from Uranium Recovery Operations., Strenge, D.L.; Bander, T.J., April 1981, NTIS; GPO \$5.50

ORNL-5711

The BOLD VENTURE Computation System for Nuclear Reactor Core Analysis, Version III., Vondy, D.R.; Fowler, T.B.; Cunningham, G.W., III, June 1981, NTIS, PC A15/MF A01

ORNL/TM-7818

Neutron-Photon Multigroup Cross Sections for Neutron Energies Less than or Equal to 400 MeV., Alsmiller, R.G., Jr.; Barish, J., June 1981, NTIS, PC A03/MF A01

ORNL/TM-7839

Final Analysis of the GCFR Exit Shield Integral Experiment., Ingersoll, D.T.; Cramer, S.N., July 1981, NTIS, PC A04/MF A01

PPPL-1791

Fusion Energy Calorimeter for the Tokamak Fusion Test Reactor., Jassby, D.L.; Imel, G.R., April 1981, NTIS, PC A02/MF A01

RIJNHUIZEN-81-129

On Fusion and Fission Breeder Reactors; The Iiasa Report RR-77-8 Reviewed and Updated., Brandt, B.; Klippel, H.Th.; Schuurman, W., February 1981, Stichting voor Fundamenteel Onderzoek der Materie, Jutphaas, Netherlands, Instituut voor Plasmafysica

RSS-TM-17

Neutronic Analysis of the Three Mile Island Unit 2 Ex-Core Detector Response., Malloy, D.J.; Chang, Y.I., June 15, 1981, Applied Physics Division, Argonne National Laboratory, 9700 South Cass Avenue, Argonne, IL 60439

Atomic Data and Nucl. Data Tables, 25(2), 91-184Energy Levels for the Configurations $(3d + 4s)^n 4p$ in the First Spectra of the Iron Group, K I-Ga I., Roth, C., March 1980

Atomkernenergie, 30(3), 219-225A Series Solution of the Double-P_L Equations for a Sphere., Hauptmanns, U., 1977

Atomkernenergie, 33(3), 195-198 Neutron Thermalization in Heavy Water. Measurement and Calculation of Spectra., Abbate, M.; Lolich, J., 1979 Health Phys., 36(2), 186-188 Simplified Calculation of a Barrier Thickness for Scattered Radiation from Cobalt-60., Miller, T.M.; Cember, H., February 1979

Health Phys., 41(1), 184-187 Kerma Transmission Through Various Materials for a p(66)Be(49) Neutron Beam., Awschalom, M.; Hrejsa, A.; Rosenberg, I.; Ten Haken, R.K., July 1981

J. of Fusion Energy, 1(1), 87-101 Radiological Aspects of Fusion Reactor Safety: Risk Constraints in Severe Accidents., Kazimi, M.S.; Sawdye, R.W., January 1981

J. Nucl. Sci. Technol., 15(9), 645-657 Numerical Solution of Time-Dependent Two-Dimensional P₁ Equation of Neutron Transport., Tsuruta, T.; Ohtani, N.; Nisihara, H., September 1978

Med. Phys., 6(2), 149-152 Computer Generated Scatter Dose Distributions for 6-MV Radiotherapy Photon Beams., Bloch, P.; Wallace, R., March 1979

Nucl. Technology, 54(2), 180-200 Calculation of Neutron Source Strength in Fast Flux Test Facility Fuel as a Function of Irradiation., Ombrellaro, P.A.; Johnson, D.L., August 1981

Nucl. Technology/Fusion, 1(3), 302-358 An Overview of Inertial Fusion Reactor Design., Monsler, M.J.; Hovingh, J.; Cook, D.L.; Frank, T.G.; Moses, G.A., July 1981

Nucl. Safety, 22(2), 205-213 Methodologies for Calculating the Radiation Dose to Man from Environmental Releases of Tritium., Till, J.E.; Etnier, E.L.; Meyer, H.R., March-April 1981

Nucl. Sci. Eng., 78(2), 175-177 Monte Carlo Self-Shielding Corrections for Use with Neutron Spectrum Unfolding Codes. (Tech. Note), Brenner, D.J., June 1981

Nucl. Sci. Eng., 78(2), 178-182 Fast Neutron Spectrum in Lithium with a 14-MeV Neutron Source. (Tech. Note), Profio, A.E.; Shani, G.; Dissanaike, G.A., June 1981 Nucl. Sci. Eng., 78(2), 185-190

Decay Rate and Decay Heat Data from Fission Products. (Tech. Note), Shure, K., June 1981

Nucl. Sci. Eng., 78(3), 259-272

Calculation of Neutron and Gamma-Ray Energy Spectra for Fusion Reactor Shield Design: Comparison with Experiment., Santoro, R.T.; Alsmiller, R.G., Jr.; Barnes, J.M.; Chapman, G.T., July 1981, Announced previously as ORNL/TM-7360

Nucl. Sci. Eng., 78(3), 273-283

Transport Calculation of Gamma Rays Including Bremsstrahlung by the Discrete Ordinates Code PALLAS., Takeuchi, K.; Tanaka, S.; Kinno, M., July 1981

Nucl. Sci. Eng., 78(3), 284-293 One-Dimensional Discrete Ordinates Adjoint Calculations of Cylindrical Target Responses to Plane-Incident Sources., Dupree, S.A.; Morel, J.E., July 1981

Nucl. Sci. Eng., 78(3), 297-304 A Study of Neutron Flux and Source Approximations in Slab Geometry. (Tech. Note), Wasastjerna, F., July 1981

Part. Accel., 9(3), 201-206 Data for Simple Estimates of Shielding Against Neutrons at Electron Accelerators., Tesch, K., March 1979

Physics Data, 17(1), 1-249 Compilation of Experimental Values of Internal Conversion Coefficients and Ratios for Nuclei with Z Equals to or Less than 60. Hansen, H.H., 1981

Radiation Res., 77(1), 21-33 Depth-Dose Relationships near the Skin Resulting from Parallel Beams of Fast Neutrons., Chen, S.Y.; Chilton, A.B., January 1979

X-Ray Spectrom., 8(2), 85-91 New Basic Empirical Expression for Computing Tables of X-Ray Mass Attenuation Coefficients., Thinh, T.P., April 1979

COMPUTER CODES LITERATURE

- AERE-R 9601 (Rev.) MONTUK Description of the Transmutation and Activation Data Library UKCTRIIIA., Jarvis, O.N., Nuclear Physics Division, AERE, Harwell, UK, May 1980
- CONF-8010113, 19-33 EURLIB-4 Checking and Improvements of the Multigroup Data Library EURLIB-4., Hehn, G.; Mattes, M.; Al Malah, K.; Kicherer, G., IKE University, Stuttgart, West Germany, October 1980
- CONF-8010113, 145-152 DUCKPOND The Application of DUCKPOND to the Oak Ridge PCA Calculational Blind Test., Hall, M.C.G.; McCracken, A.K.; Packwood, A., Reactor Physics Division, AEE Winfrith, UK, October 1980
- CONF-8010113, 205-220 DUCK POND DUCK POND - A Perturbation Monte Carlo and Its Applications., Hall, M.C.G., Reactor Physics Division, AEE Winfrith, UK, October 1980
- Health Physics, 37(1), 145-159 FANTOM Monte Carlo Calculations of the Response of an External Detector to a Photon Source in the Lungs of a Heterogeneous Phantom., Bhati, S.; Sharma, R.C.; Somasundaram, S., Health Physics Division, Low-Level Counting Laboratory, BARC Hospital, Bombay 400094, India, July 1979

INER-0311 HUNDUN;COLAC;CORRP;TRITIUM;TRIP;WASTE

- PWR Outside Core Radiation Source Evaluation Using EMERS Code Package., Yang, S.-C.; Su, M.-F., Institute of Nuclear Energy Research, Taiwan, February 1980
- IRI-131-77-05; THD-H-RF-146 ETOF; FOCUS ETOF - A Program to Prepare a Cross-Section Data Tape from the ENDF/B File for the Adjoint Monte Carlo Code FOCUS., Hoogenboom, J.E., Interuniversity Reactor Institute, Delft, The Netherlands, 1979
- KFK 2386/111 KEDAK-3
 Evaluations for the German Nuclear Data Library KEDAK-3. Part 2: Fissile and Fertile Materials., Goel, B.; Weller, F., KFK Karlsruhe, West Germany, March 1977

- ORNL/CSD/TM-82 PIE PIE - A FORTRAN Subroutine for Plotting Circles, Concentric Circles, Pie Charts, and Shaded Pie Charts., Dillard, C.A., Oak Ridge National Laboratory, TN, October 1979
- ORNL-TM-7097 DOT IV The Stability of the Mesh-Cornered Synthetic Method of Diffusion Acceleration of the DOT IV Transport Code., Aull, J., Oak Ridge National Laboratory, TN, October 1979
- ORNL/TM-7389 PUFF 2 Multigroup Covariance Matrices for Fast Reactor Studies., Smith, J.D.; Broadhead, B.L., Oak Ridge National Laboratory, TN
- ORNL/TM-7631 .. NEUTRON ENERGY SPECTRA Method for Sampling from Fission Neutron Energy Spectra., Froehner, F.H.; Spencer, R.R., Oak Ridge National Laboratory, TN, February 1981
- PNCT843-80-11; PNCT841-78-01(Rev.) ACRO ACRO - A Computer Program for Calculating Organ Doses from Acute or Chronic Inhalation and Ingestion of Radionuclides., Kitahara. Y.: Kishimoto, Y.; Shiohara, K., Power Reactor and Nuclear Fuel Development Corp., Japan, April 1980
- PNL-SA-7352, 429-435 (Vol.2) ARDISC ARDISC Model: A Computer Program to Calculate Distribution of Trace Element Migration in Partially-Equilibrating Media., Strickert, R., Battelle Pacific Northwest Labs., Richland, WA, 1979
- RISO-R-392 GFX-GAMIX Program System for Computation of the Terrestrial Gamma-Radiation Field., Kirkegaard, P.: Lovborg, L., Riso National Laboratory, Denmark, February 1979
- RRA-N7914 QADMOD-G Utilization Instructions for QADMOD-G., Price, J.H.; Blattner, W.G.M., Radiation Research Associates, Fort Worth, TX, December 1979
- Trans. Am. Nucl. Soc., 33, 708-710 BONE MARROW Calculation of Neutron Kerma Factors and the Related Cellular Survival Parameters for Bone Marrow., Chen, S.Y.; Kaul, D.C., Science Applications, Inc., Schaumburg, IL, 1979
- TRG Memorandum 7506(R) CYLDOS
 CYLDOS A Cylindrical Geometry Gamma-Ray
 Flux Attenuation Code., Richardson, E.I.,
 United Kingdom Atomic Energy Authority, Risley,
 Warrington, UK, April 1977

TRG Report 2692(R) CRACKLE CRACKLE: A Computer Code for CFR Fuel Management Calculations., Burstall, R.F.; Ball, M.A.; Thornton, D.E.J., United Kingdom Atomic Energy Authority, Risley, Warrington, UK, December 1975

.