

RSIC Newsletter



RADIATION SHIELDING INFORMATION CENTER

OAK RIDGE NATIONAL LABORATORY

OPERATED BY UNION CARBIDE CORPORATION FOR THE DEPARTMENT OF ENERGY

POST OFFICE BOX X •
OAK RIDGE, TENNESSEE 37830

No. 196

April 1981

The journey of a thousand miles starts with a single step. — Chinese Proverb

HELP SOUGHT TO CLARIFY RESEARCH AREAS OF RSIC USERS

RSIC interactions with its user community is made possible through sponsorship from four funding agencies. These include the Department of Energy (DOE), Division of Reactor Research and Technology and Office of Fusion Energy, the Nuclear Regulatory Commission, and the Defense Nuclear Agency. Each sponsor shares in the cost of RSIC operations and attempts, although it is not always possible, to provide enough funding to carry a fair share of the total operating costs.

We have found that RSIC is a technology resource which has benefited users doing research in areas supported by all our funders. In recent years, since the acquisition and operation of our Data General Eclipse minicomputer, we have developed means of getting an approximate handle on the composition of the research areas being investigated by our user community. This information can be recorded as we process requests and later analyzed using an internally developed data base management and statistical analysis program, ADES. To get a more accurate picture, we assign a research profile for each RSIC user. For example, if the requester tells us that, in general, he spends 30% of his time on breeder reactor work, 40% on fusion, and 30% on light water reactors, we count a request from him using those percentages. If a request is accompanied by a specific statement saying that it will be used, for example, in a breeder project, it will be counted as 100% breeder.

To help us get a better handle on our research profile, each RSIC user is asked to provide us with a research profile. To guide your thinking, we have appended to this issue of the *RSIC Newsletter* a form for your use and will also include this form in shipping requested material to you. Please fill out and return it promptly. In addition, if a particular request can be isolated as to its application, the user should so state in the letter of request or in the conversation with an RSIC staff member.

Please help us to help you.

NEWS OF RSIC PRODUCTS REVIEW

The internal audit of RSIC products continues with current emphasis on the Peripheral-to-Shielding Routines (PSR). A critical review of the code packages PSR 51—62 resulted in the removal of three from the collection as follows:

PSR-51/SMUG

The SMUG module in PSR-63/AMPX II is recommended as being current state-of-the-art.

PSR-52/MACK

The latest technology is packaged as PSR-132/MACK-IV and is recommended.

PSR-61/LAPHAN

ENDF/B-III vintage. LAPHANGAS module of PSR-63/AMPX-II is recommended.

CHANGE IN THE COMPUTER CODE COLLECTION

The following addition was made in March.

PSR-151/CHENDF

The collection of handling codes for ENDF/B-V data has been augmented by the addition of RESEND5 which was contributed by Oak Ridge National Laboratory. The current code package contains RIGEL5, STNDRD, CRECT, CHECK5, FIZCN, and INTEND. The codes were originally developed at Brookhaven National Laboratory and were made operational and, in some cases, improved and upgraded at ORNL. References: User Input Instructions, informal notes. Earlier versions are documented in ENDF-110, *Description of the ENDF/B Processing Codes and Retrieval Subroutines*. IBM-370/3033.

CHANGE IN THE DATA LIBRARY COLLECTION

The following change was made in March.

DLC-64/UKCTRI-81

The 46 neutron cross-section library for fusion reactor calculations has been updated by the original contributor, the University of Birmingham in England, with the addition of kerma factors and activation cross sections. A slightly modified version of the retrieval code GETEX, which can convert the data to ANISN format, is also supplied. In addition to the kerma and activation data, the library includes cross sections for H, H-gas, D, D-gas, T, Li-6, Li-7, Be-9, B-10, B-11, C, O, F, Na, Al, K, Ti, V, Cr, Fe, Ni, Cu-63, Cu-65, Zr, Nb, Mo, and Pb. Reference: University of Birmingham, Department of Physics, Paper No. 79-02, *The UKCTRI Data Library: 46-Group Neutron Cross Sections for Fusion Reactor Calculations*, T. D. Benyon and N. P. Taylor, and University of Birmingham, Department of Physics, Paper No. 80-01, *46-Group Neutron Heating Factors for the UKCTRI Data Library*, N. P. Taylor. IBM-370/3033.

NUCLEAR STANDARDS NEWS

The ANS Standards Steering Committee met on February 26, 1981. New project charters approved were:

Project	Chairman
ANS-6.4.2 Specification for Radiation Shielding Materials	E. A. Normand (Northwest Energy Service Co.)
ANS-6.4.3 Gamma-Ray Attenuation Coefficients and Buildup Factors for Engineering Materials	D. K. Trubey (Oak Ridge National Laboratory)
ANS-8.19 Administrative Practices for Nuclear Criticality Safety	D. R. Smith (Los Alamos National Laboratory)

In addition, the SSC approved the efforts of NUPPSCO to develop one or more proposed standards on emergency response facilities. Specific charters are to be prepared by a task force.

ANS efforts to gain the participation of other societies to revise N101.6-1972, *Concrete Radiation Shields*, have been unsuccessful. It is felt that ANS-6.4-1977, *Guidelines on Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power Plants*, and ACI-349-76, *Code Requirements for Nuclear Safety Related Structures*, each cover a substantial part of the material in N101.6 in a more current fashion and are utilized in present engineering practice. ANS has advised the Nuclear Standards Management Board that it will no longer sponsor the standard and that it should be withdrawn.

WORKING GROUP ON BENCHMARK PROBLEMS REQUESTS SOLUTIONS TO RADWASTE FACILITY PROBLEM

The American Nuclear Society Working Group on Shielding Benchmarks, ANS-6.2.1, requests solutions to a gamma-ray nuclear radwaste facility problem. They would like to have solutions by all applicable computing methods from semi-analytical to Monte Carlo. The specifications are available from RSIC upon request.

PURCHASE OF DOE MICROFICHE

A private vendor has been contracted to handle distribution of Department of Energy reports in microfiche. The announcement was made by James G. Smith, Director, Microphotography and Distribution Division, DOE/TIC, in a 12/18/80 letter sent to all current or prospective purchasers of DOE microfiche. The letter stated that DOE has awarded a contract to Engineered Systems, Inc., to manufacture and sell copies of DOE research and development reports until December 31, 1985. Microfiche will be manufactured from diazo film and will have a reduction ratio of 24 times.

Information categories and estimated prices for the subscription year were described in the order form enclosed with the letter. Unit prices for microfiche will be \$0.079 when all categories are purchased; \$0.152 per fiche if selectively purchased by category; and \$0.50 per fiche if individually ordered. Engineered Systems, Inc., will also sell hard copy enlarged from the master negative for \$0.15 per page of original text. Blank pages in certain instances will be included and charged for at the stipulated rate, because the enlargement process requires that methods involved in the original filming be followed. Postage and sales tax (if applicable) will be a separate cost. For organizations subscribing to all categories, first class postage and handling will be about \$800.00 for the period January 1, 1981, through December 31, 1981. If the purchaser is a university or public library, postage and handling will be about \$60.00 for the same period. All financial arrangements must be made with Engineered Systems, Inc. Payment is due within 30 days from date of invoice.

Those wishing to purchase DOE microfiche, or needing details about information categories and estimated prices can contact: Engineered Systems, Inc., P. O. Box 866, Oak Ridge, TN 37830.

EIA DATA INDEX: AN ABSTRACT JOURNAL

Users of the Federal Energy Data Index, or FEDEX data base on RECON, now have access to abstracting and indexing at the table-and-graph level through a new Energy Information Administration (EIA) publication: *EIA Data Index: An Abstract Journal*, DOE/EIA-0233(80). The first issue was published December 1980.

Over 1800 unique tables and graphs are printed in EIA periodicals each year. The *EIA Data Index* contains abstracting and indexing for tables, graphs, and other formatted data reported in EIA publications. It includes a complete subject index and a report number listing for all EIA publications as well as complete information on how to order these publications.

The *EIA Data Index: An Abstract Journal* is a companion volume to the *EIA Publications Directory: A User's Guide*, which provides abstracts and indexes to all EIA publications at the document level. Both publications are generated from the FEDEX data base, which has been developed by the EIA in cooperation with TIC. The *EIA Data Index* will be updated semi-annually. The *EIA Publications Directory* is issued quarterly and cumulated annually. Both are available from the U. S. Government Printing Office.

For information on availability of these and other EIA publications, contact: U. S. Department of Energy, Energy Information Administration, National Energy Information Center, EI-72, Forrestal Building, Washington, DC 20585 (202/252-8800 or FTS 252-8800).

UPCOMING CONFERENCES

Radiation Protection Symposium in Scotland

The Call for Papers has been issued by the Society for Radiological Protection for its Third International Symposium on "Radiation Protection – Advances in Theory and Practice" to be held in Inverness, Scotland on June 6–11, 1982. The scientific program will cover a broad spectrum of radiological protection topics embracing both theoretical and practical aspects. Authors wishing to present papers are invited to submit abstracts for consideration. The deadline for these abstracts is July 15, 1981.

Suggested topics for the conference are: radiation and risks, radiobiology and epidemiology, instrumentation and equipment, design of facilities, radioactivity in the environment, education and public education, legal and regulatory aspects, accident management, non-ionizing radiation, and medical techniques. The proceedings will be published.

Further information concerning the conference may be addressed to **G. C. Roberts**, Secretary, Third SRP International Symposium, National Radiological Protection Board, Harwell, Didcot, Oxfordshire OX11 0RQ, England.

1982 Ionizing Radiation Symposium Planned

The Call for Papers and Scientific Exhibits has been issued for the "International Symposium on Applications and Technology of Ionizing Radiations" to be held March 12–17, 1982, at the University of Riyadh, Riyadh, Saudi Arabia. Of particular interest are papers which *survey* an area of technology or aspect of the following topics (as compared to papers concerned with very detailed points): Medical Applications, Radiation Protection, High Level Gamma Irradiation, Radiation Chemistry, Radioisotope Production, Industrial and Agricultural Applications, and Dosimetry.

Copy-ready abstracts must be submitted by September 15, 1981. Address all correspondence to: **Raymond C. Barrall**, Cancer Therapy Institute, King Faisal Hospital, Box 3354, Riyadh, Saudi Arabia. All correspondence *must* be sent by *overseas* air mail.

1982 Reactor Dosimetry Symposium

A Call for Papers has been issued for the "4th ASTM-EURATOM Symposium on Reactor Dosimetry" to be held at the National Bureau of Standards, Washington, DC, March 22–26, 1982. The Symposium is sponsored by Petten Establishment of the Joint Research Centre of the Commission of the European Communities, ASTM Committee E10 on Nuclear Technology and Applications, U. S. Nuclear Regulatory Commission (NRC), the Electric Power Research Institute (EPRI), all in cooperation with the International Atomic Energy Agency (IAEA).

The theme of the symposium encompasses techniques, data bases, and standardization. The focus will be on the application and requirements for radiation metrology of irradiated fuels and materials in fission and fusion technology. Papers are solicited for the following topics (involving light water reactors, fast breeder reactors, and fusion systems) as well as related subjects: Characterization of Environments, Irradiation Monitoring of Experiments, Adjustment Codes and Uncertainties, Benchmark Fields and Calibration Procedures, Nuclear Data Needs and Problems, Metrology Techniques (new developments and improvements), Fuel Cycle Dosimetry, Radiation Damage Correlations of Structural Materials and Damage Analysis Techniques, Nuclear Heating and Gamma-Ray Dosimetry, and Neutron and Gamma-Ray Transport Calculations.

The Symposium will be organized into formal presentations, a poster session and workshops. Authors who wish to submit summaries for publication in the proceedings without presentation, or those who wish to present their material in the poster session, should announce this.

Information concerning the Symposium may be secured from: **J. K. Schmotzer**, Babcock & Wilcox, Lynchburg Research Center, P. O. Box 1260, Lynchburg, VA 24505, USA; **H. Rottger**, Joint Research Centre, Petten Establishment, HFR Division, Postbus 2, 1755 ZG Petten (N. H.), The Netherlands; or **F. B. K. Kam**, Oak Ridge National Laboratory, P. O. Box X, Oak Ridge, TN 37830, USA.

The original is due March 19, 1982.

VISITORS TO EPIC

The following persons came for an orientation visit and/or to use EPIC facilities during the month of March: **James F. Baur**, General Atomic Company, San Diego, California; **D. V. Gopinath**, Reactor Research Centre, India; **Arnost Honig**, BRNO Technical University, Czechoslovakia; **Johnny Rosen**, Nuclear Energy Agency, France; **Y. Kato**, **Kenji Sumita** and **Kazutaka Kawamura**, Osaka University, Japan; **Frank Sweeney** and **David D. Yue**, Oak Ridge National Laboratory, Oak Ridge, Tennessee; **Uwe Trinks**, Technical University of Munich, W. Germany; **Maurice Wilkenson**, Boeing Aerospace Corp., Seattle, Washington; **John G. Williams**, University of Arkansas and University of London, Fayetteville, Arkansas, and United Kingdom; **Roman Zurba** and **John Blansche**, Atomic Energy of Canada, Mississauga, Canada.

MARCH ACCESSION OF LITERATURE

The following literature cited has been ordered for review, and that selected as suitable will be placed in the RSIC Information Storage and Retrieval Information System (SARIS). This early announcement is made as a service to the shielding community. Copies of the literature are not distributed by RSIC. They may generally be obtained from the author or from a documentation center such as the National Technical Information Service (NTIS), Department of Commerce, Springfield, Virginia 22151.

RSIC maintains a microfiche file of the literature entered into SARIS, and duplicate copies of out-of-print reports may be available on request. Naturally, we cannot fill requests for literature which is copyrighted (such as books or journal articles) or whose distribution is restricted.

THIS LITERATURE IS ON ORDER. IT IS NOT IN OUR SYSTEM. PLEASE ORDER FROM NTIS OR OTHER AVAILABLE SOURCE AS INDICATED.

REACTOR AND WEAPONS RADIATION SHIELDING LITERATURE

ANL/FPP-80-1, Vol. I and II
STARFIRE - A Commercial Tokamak Fusion Power Plant Study., Baker, C.C.; Abdou, M.A.; DeFreece, D.A.; Trachsel, C.A.; Graumann, D.W.; Kokoszinski, J.; Barry, K., September 1980, NTIS, PC A99/MF A01

BNL-28480; CONF-800950-15
Design Studies of an Aluminum First Wall for Intor., Powell, J.R.; Fillo, J.A.; Yu, W.S.; Hsieh, S.Y.; Pearlman, H.; Kramer, R.; Franz, E.; Craig, A.; Ferrell, K., 1980, NTIS, PC A02/MF A01

CEA-CONF-4735; CONF-791051-26
Damage/Activation Ratio Measurements by Tungsten Dosimetry in Core and Periphery of Julich FRJ-2 Reactor., Alberman, A.; Thierry, M.; Schneider, W.; Weise, L., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CEA-CONF-4739; CONF-791051-25 (In French)
Required and Obtained Accuracy in the Dosimetry of Nuclear Reactor Pressure Vessel Steels., Alberman, A.; Genthon, J.P.; Mas, P.; Perdreau, R., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CEA-CONF-4741; CONF-791051-24 (In French)
DOMPAC Dosimetry Experiment. Neutronic Simulation of the Thickness of a PWR Pressure Vessel. Irradiation Damages., Alberman, A.; Faure, M.; Thierry, M.; Hoclet, O.; Le Dieu de Ville, A.; Nimal, J.C.; Soulat, P., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CEA-CONF-4752; CONF-790974-3
DOMPAC: Neutronic Model and Experimental Damage. Characterization of a LWR - Pressure Vessel., Alberman, A.; Nimal, J.C., 1979, NTIS (U.S. Sales Only), PC A02/MF A01

CEGB-RD/B/N-4661

Computer Model for Predicting Particulate Transport in Reactor Coolant Geometries., Reeks, M.W., October 1979, NTIS (U.S. Sales Only), PC A03/MF A01

CONF-790974

CAPRICE 79: Correlation Accuracy in Pressure Vessel Steel as Reactor Component Investigation of Change of Material Properties with Exposure Data., Schneider, W., Proceedings of an IAEA Technical Committee Meeting held in the KFA Julich, 24-27 September 1979., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.162-174

Experience with Neutron Spectrum Unfolding Codes., Zijp, W.L., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.175-191

Experience with Damage to Activation Ratios., Zijp, W.L.; Nolthenius, H.J., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.192-209

Uncertainties and Biases Arising from Methods Approximations., Maerker, R.E.; Wagschal, J.J., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.210-227

Towards an Adequate Evaluation of LWR Pressure Vessel Steel Irradiation Exposures., Fabry, A.; Kam, F.B.K., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.228-232

The IAEA International Reactor Dosimetry File(IRDF)., Kocherov, N.; Muir, D.W., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.233-248

Special Activities at the IAEA on Intercomparison and Errors Involved in the Application of Neutron Spectral Unfolding., Ertek, C., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.249-270

DOMPAC: Neutron Model and Experimental Damage Characterization of a LWR Pressure Vessel., Alberman, A.; Nimal, J.C., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.288 (Abstract Only)

Review of Steel Embrittlement Investigation Results with Comments on Data Scatter and Uncertainties., Brumovsky, M., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.310-333

Neutron Exposure Dependence of the Embrittlement of Reactor Pressure Vessel Steels: Correlation Models and Parameters., Odette, G.R., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.334-346

A Review of Correlation Data., Weise, L., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.347-374

The Influence of the Neutron Spectrum on the Embrittlement of Steels for Reactor Vessels., Mas, P.; Perdreau, R.; Tran-Dai, P., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.375-383

Reevaluation of Ferritic Steel Ductile-Brittle Transition Temperature Change Data Used in Damage Function Analysis., Simons, R.L., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.386-388

Basic Considerations in the Irradiation Surveillance Program in Switzerland., Njo, D.H.; Varga, T.; Ullrich, G., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.389-410

Behaviour of Neutron Irradiation Embrittlement of Mn-Mo-Ni Steels for LWR Pressure Vessel Ranging from Low to High Fluences., Kodaira, T.; Miyazono, S.; Ishimoto, K.; Nakajima, N.; Ohtsuka, T., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.411-438

U.S. Nuclear Reactor Vessel Integrity Considerations., Marston, T., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

CONF-790974, pp.439-472

Surveillance Irradiations and Reactor Pressure Vessel Safety., Nagel, G., May 1980, ZENTRALBIBLIOTHEK der Kernforschungsanlage Julich GmbH, Postfach 1913, D-5170 Julich (Germany)

EPRI-NP-926

OZMA - A Code to Calculate Resonance Reaction Rates in Reactor Lattices Using Resonance Profile Tabulations., Barhen, J.; Rothenstein, W., February 1981, Research Reports Center (RRC), Box 50490, Palo Alto, CA 94303, Tel.(415) 965-4081

EPRI-NP-1634

Improvement of the SAM-CE Criticality Capability and Analysis of Thermal Reactor Benchmarks., Lichtenstein, H.; Troubetzkoy, E.S., November 1980, NTIS

EPRI-NP-1658

Integral Tests of Fast Neutron Dosimetry Cross Sections., Johnson, R.H., January 1981, Research Reports Center (RRC), Box 50490, Palo Alto, CA 94303, Tel.(415) 965-4081

FDA-80-8129

Possible Genetic Damage from Diagnostic X-Irradiation. A Review., Withrow, T.J.; Andersen, F.A.; Yao, K.T.S.; Stratmeyer, M.E., August 1980, Department of Health and Human Services, Rockville, MD

HEDL-SA-2215-FP; CONF-801107-46

Development of a Standard for Computer Program Verification and Control., Dunn, T.E.; Ozer, O., 1980, NTIS, PC A02/MF A01

INDC(SEC)-74/LNQ

1980 Compilation of National Nuclear Data Committees., IAEA, April 1980, NTIS (U.S. Sales Only), PC A03/MF A01

INDC(NDS)-112/LNQ

Compilations and Evaluations of Nuclear Structure and Decay Data. Issue No.5., Lorenz, A., No Date, NTIS (U.S. Sales Only), PC A02/MF A01

INFCE/DEP/WG-5/10

Role of Fast Breeders in Japan., Oyama, A.; Tomabechi, K., September 1978, NTIS (U.S. Sales Only), PC A02/MF A01

INFCE/DEP/WG-8/73

Symbiosis of Near Breeder HTR's with Hybrid Fusion Reactors., Seifritz, W., July 1978, NTIS (U.S. Sales Only), PC A02/MF A01

INIS-mf-4645, pp.342-349 (In Russian)

Estimation of Spectral Distributions of ¹³⁷Cs Gamma Radiation Scattered in Thin Flat Radiators., Kheteev, M.V.; Stavitskij, R.V., 1975, Vsesoyuznyj Nauchno-Issledovatel'skij Inst. Radiatsionnoj Tekhniki, Moscow (USSR)

LBL-11297; CONF-8005125-1

Health Effects of the Nuclear Accident at Three Mile Island., Fabrikant, J.I., May 1980, NTIS, PC A02/MF A01

NUREG/CR-1872; ORNL/NUREG/TM-428

Reactor Calculation "Benchmark" PCA Blind Tests Results., Stallman, F.W.; Kam, F.B.K.; Eastham, J.F.; Baldwin, C.A., January 1981, NTIS

NRPB-DL3

Dosimetric Quantities and Basic Data for the Evaluation of Generalised Derived Limits., Harrison, N.T.; Simmonds, J.R., December 1980, HMSO, ISBN 0 85951 146 4

NRPB-R109

A Model to Calculate Exposure from Radioactive Discharges into the Coastal Waters of Northern Europe., Clark, M.J.; Grimwood, P.D.; Camplin, W.C., November 1980, HMSO, ISBN 0 85951 140 5

ORNL-5316; ENDF-291

A User's Manual for the FORSS Sensitivity and Uncertainty Analysis Code System., Lucius, J.L.; Weisbin, C.R.; Marable, J.H.; Drischler, J.D.; Wright, R.Q.; White, J.E., January 1981, NTIS, PC A16/MF A01

ORNL/CSD/TM-135

Light Water Reactor-Pressure Vessel Surveillance Project Computer System., Merriman, S.H., October 1980, NTIS, PC A03/MF A01

ORNL/TM-6495

Comparison of the COMRADEX-IV and AIRDOS-EPA Methodologies for Estimating the Radiation Dose to Man from Radionuclide Releases to the Atmosphere., Miller, C.W.; Hoffman, F.O.; Dunning, D.E., Jr., January 1981, NTIS, PC A03/MF A01

ORNL/TM-7526

Estimated Nuclear Effects in the Neutral Beam Injectors of a Large Fusion Reactor., Lillie, R.A.; Santoro, R.T.; Alsmiller, R.G., Jr., December 1980, NTIS, PC A03/MF A01

ORNL/TM-7528

A Phenomenological Model for Particle Production from the Collisions of Nucleons and Pions with Fissile Elements at Medium Energies., Alsmiller, F.S.; Alsmiller, R.G., Jr.; Gabriel, T.A.; Lillie, R.A.; Barish, J., March 1981, NTIS, PC A05/MF A01

ORNL/TM-7562

Overview of EBT Reactor Projections., Uckan, N.A., December 1980, NTIS, PC A04/MF A01

ORNL/TM-7597

Expected Environments for a Defense High-Level Waste Repository in Salt., Rickertsen, L.D.; Claiborne, H.C., March 1981, NTIS, PC A04/MF A01

ORNL/TM-7705

Neutron and Gamma Ray Streaming Calculations for the ETF Neutral Beam Injectors., Lillie, R.A.; Santoro, R.T.; Alsmiller, R.G., Jr.; Barnes, J.M., February 1981, NTIS, PC A03/MF A01

PB-80-170384

Estimated Target Accuracies for Neutronic Responses in Fusion Reactors., Verschuur, K.A., December 1979, NTIS, PC A02/MF A01

PPPL-1741

Penetration Seals for TFTR Shielding., Hondorp, H.L., December 1980, NTIS \$4.00

RPC-80-003

Report of the Task Force on Occupational Radiation Exposure Regulations., U.S. Radiation Policy Council, August 15, 1980, U.S. Government Printing Office - 1981 723-979/793

STI/PUB/521; CONF-790316

Application of the Dose Limitation System for Radiation Protection - Practical Implications., Proceedings of a Topical Seminar on the Practical Implications of the ICRP Recommendations (1977) and the Revised IAEA Basic Standards for Radiation Protection. Held in Vienna, 5-9 March 1979., IAEA, 1979, Unipub, \$78.00

SMRE-Trans-6814

Reduction of the Residual Risk in PWR Nuclear Power Plants., Smidt, D., May 1976, Health and Safety Executive, Research and Laboratory Services Division, London, England

UCRL-50400, Vol.23

ENSL and CDRL: Evaluated Nuclear Structure Libraries., Howerton, R.J., February 2, 1981, NTIS, PC \$5.00/MF \$3.50

Atomkernenergie, 32(3), 169-174

Gamma Ray Penetration in Heterogeneous Systems., Williams, M.M.R., 1978

Health Phys., 40(2), 205-219

Broad Beam Attenuation in Leco for 50-140 kVp X-Rays., Wohni, T., February 1981

Health Phys., 40(2), 219-222

Fusion Reactor Neutron Dosimetry., Barton, T.P.; Easterly, C.E.; Zeimer, P.L., February 1981

Int. J. Appl. Radiat. Isot., 29(7), 419-422

The Calculation of Gamma-Rays Albedo by the Monte Carlo Method., Seda, J.; Kluson, J.; Cechak, T., July 1978

J. Nucl. Sci. Technol., 15(6), 400-410

Improvement of Correlated Sampling Monte Carlo Methods for Reactivity Calculations., Nakagawa, M.; Asaoka, T., June 1978

J. Nucl. Sci. Technol., 16(5), 377-379

Absolute Fission-Rate Distribution in Graphite-Reflected Lithium Oxide Blanket Assembly., Maekawa, H.; Oyama, Y.; Kusano, J.; Nakamura, T., May 1979

Nucl. Instrum. Methods, 155(1-2), 249-252

Anisotropic Neutron Emission from Laboratory Sources., Rao, U.S.; Kumar, A.; Misra, S.C.; Nagarajan, P.S.; Gupta, U.C., September 15, 1978

Nucl. Sci. Eng., 77(1), 1-2

Integral Transport Theory Formalism for Diffusion Coefficient Calculations in Wigner-Seitz Cells., Benoist, P., January 1981

Nucl. Sci. Eng., 77(1), 13-19

Interface and Source Problems by a Transport Theoretic P_N Approximation., Gangly, K.; Sengupta, A., January 1981

Nucl. Sci. Eng., 77(1), 40-50

Progress in Integral Data and Their Accuracy: Average Neutron Cross Sections in the Californium-252 Benchmark Field., Mannhart, W., January 1981

Nucl. Sci. Eng., 77(1), 71-83

Computational Benchmark for Neutron Penetration in Iron., Hendricks, J.S.; Carter, L.L., January 1981

Nucl. Sci. Eng., 77(3), 285-296
Sensitivity of Light Water Reactor Fuel Cycle Parameters and Costs to Uncertainties in Thermal Nuclear Data and Methods., Ryskamp, J.M.; Harris, D.R.; Becker, M., March 1981

Nucl. Sci. Eng., 77(3), 316-343
Generalized Perturbation Theory for Nonlinear Systems from the Importance Conservation Principle., Gandini, A., March 1981

Nucl. Sci. Eng., 77(3), 351-354
Using Orthogonal Functions with a Finite Element Method for Approximating Even-Parity Neutron Flux in Slab Geometry. (Tech. Note), Splawski, B.A.; Ziver, A.K.; Galliara, J., March 1981

Nucl. Sci. Eng., 77(3), 354-359
Stability Analysis for Integro-Differential Coupled-Core Reactor Models. (Tech. Note), Podowski, M., March 1981

Nucl. Sci. Eng., 77(3), 360-367
Effectiveness of an Adaptive Acceleration Method for Inner Iterations in Some Neutron Diffusion Codes. (Tech. Note), Fujimura, T.; Matsui, Y., March 1981

Nucl. Sci. Eng., 77(3), 367-372
A Semianalytic Method for the Solution of the Neutron Transport Equation in Plane and Spherical Geometries. (Tech. Note), Fletcher, J.K., March 1981

Nucl. Technology, 52(3), 428-430
Increases in Neutron Yield of ^{239}Pu -Be(α ,n) Sources - II., Anderson, M.E., March 1981

Nucl. Technology, 52(3), 354-369
Measurement and Calculation of the Effectiveness of the Gas-Cooled Fast Breeder Reactor Grid-Plate Shield., Slater, C.O.; Cramer, S.N.; Ingersoll, D.T.; Williams, M.L.; Muckenthaler, F.J.; Manning, J.J.; Hull, J.L., March 1981

Radiation Res., 75(3), 462-470
Basic Concepts in Dosimetry. A Critical Analysis of the Concepts of Ionizing Radiation and Energy Imparted., Carlson, G.A., September 1978

Transp. Theory Stat. Phys., 7(3), 81-101
Asymptotic Equivalence of the P_1 and Diffusion Equations., Mika, J.; Stankiewicz, R., 1978

COMPUTER CODES LITERATURE

ANL-79-82 TSOAK-M1
TSOAK-M1: A Computer Code to Determine Tritium Reaction/Adsorption/Release Parameters from Experimental Results of Air-Detrutiation Tests., Land, R.H.; Maroni, V.A.; Minkoff, M., Argonne National Laboratory, Argonne, IL, 1979, AVAIL: NTIS

ANL-80-55 GIRAFFE
Theory and Use of GIRAFFE for Analysis of Decay Characteristics of Delayed-Neutron Precursors in an LMFBR., Gross, K.C., Argonne National Laboratory, Argonne, IL, July 1980, FORTRAN IV

INER-0318 ANISCAT
A User's Manual for ANISCAT: A Code to Evaluate the Reflected Dose of the Primary Shield into Reactor Cavity., Su, M.F., Institute of Nuclear Energy Research, Lung-Tan, Taiwan, April 1980

INIS-mf-5675 (In Russian) DDCS
Taking Account of Sample Finite Dimensions in Processing Measurements of Double Differential Cross Sections of Slow Neutron Scattering., Lisichkin, Yu.V.; Dovbenko, A.G.; Efimenko, B.A.; Novikov, A.G.; Smirenkina, L.D.; Tikhonova, S.I., Gosudarstvennyj Komitet po Ispol'zovaniyu Atomnoj Ehnergii SSSR, Moscow, 1979, BESM-5

JAERI 1245 ESELEM 4
A Code for Calculating Fine Neutron Spectrum and Multi-Group Cross Sections in Plate Lattice., Nakagawa, M.; Katsuragi, S.; Narita, H., Tokai Research Establishment, Japan Atomic Energy Research Institute, Tokai, Japan, July 1976

JAERI-M 8229 ORIGIN-JR
A Computer Code for Calculating Radiation Sources and Analyzing Nuclide Transmutations., Koyama, K.; Yamano, N.; Miyasaka, S., Japan Atomic Energy Research Institute, Tokai, Japan, May 1979

J. Nucl. Med., 21(4), 379-383 CAMIRD/III
CAMIRD/III: A Revised Version of the CAMIRD/II and MIRD-S Packages for Internal Dose Calculation: Concise Communication., Bellina, C.R.; Guzzardi, R., CNR Clinical Physiology Laboratory, Pisa, Italy, April 1980, FORTRAN IV, IBM 370/168

- J. Nucl. Sci. Tech., 17(7), 539-558 ENSEMBLE
Development of Discrete Ordinates SN Code in
Three-Dimensional (X,Y,Z) Geometry for Shielding
Design., Nishimura, T.; Tada, K.; Yokobori, H.;
Sugawara, A., Japan Atomic Energy Research
Institute, Tokai, Japan, July 1980
- LA-8333-MS SED
A Comprehensive Neutron Cross-Section and
Secondary Energy Distribution Uncertainty Analysis
for a Fusion Reactor., Gerstl, S.A.; LaBauve, F.J.;
Young, P.G., Los Alamos Scientific Laboratory,
Los Alamos, NM, May 1980
- LA-8498-MS SENSIT
SENSIT: A Cross-Section and Design Sensitivity
and Uncertainty Analysis Code., Gerstl, S.A.,
Los Alamos Scientific Laboratory, Los Alamos,
NM, August 1980
- ORNL-5634 HARAD
HARAD: A Computer Code for Calculating
Daughter Concentrations in Air Following the
Atmospheric Release of a Parent Radionuclide.,
Moore, R.E., Oak Ridge National Laboratory,
Oak Ridge, TN, May 1980, FORTRAN
IV, AVAIL: NTIS
- ORNL-TM-7038 FCXSEC
FCXSEC: Multigroup Cross-Section Libraries for
Nuclear Fuel Cycle Shielding Calculations., Ford,
III, W.E.; Webster, C.C.; Diggs, B.R.; Pevey, R.E.;
Croff, A.G., Oak Ridge National Laboratory,
Oak Ridge, TN, May 1980
- ORNL/TM-7384 SPEC-4
User's Guide for the Revised SPEC-4 Neutron
Spectrum Unfolding Code., Johnson, J.O.;
Ingersoll, D.T., Oak Ridge National Laboratory,
Oak Ridge, TN, August 1980
- ORNL/TM-7485 SAMMY
User's Guide for SAMMY: A Computer Model for
Multilevel R-Matrix Fits to Neutron Data Using
Bayes' Equations., Larson, N.; Perey, F.; Harvey,
J.A., Oak Ridge National Laboratory, Oak Ridge,
TN, November 1980
- RD/B/N4888 POPFOOD
POPFOOD - A Computer Code for Calculating
Ingestion Collective Doses from Continuous
Atmospheric Releases., Nair, S.; Hotson, J.;
Stacey, A., Berkeley Nuclear Laboratories,
England, July 1980
- RD/B/N4911 POPFOOD
A User's Guide to the POPFOOD Computer Code
for Evaluating Ingestion Collective Doses., Nair,
S.; Palamountain, J., Berkeley Nuclear
Laboratories, England, September 1980,
FORTRAN IV (Level H), IBM 370/165
- Trans Am. Nucl. Soc., 33, 413-415 SHIELD
Shield System., Finch, D.R.; Chandler, J.R.;
Church, J.P., Savannah River Laboratory, Aiken,
SC, 1979
- UCRL-50400, (V.17)(Pt.C) RECENT
Program RECENT (Version 79-1): Reconstruction
of Energy-Dependent Neutron Cross Sections from
Resonance Parameters in the ENDF/B Format.,
Cullen, D.E., Lawrence Livermore Laboratory,
Livermore, CA, October 1979, AVAIL: NTIS
- UCRL-50400, V.22 GAMIDENT
GAMIDENT: A Program to Aid in the
Identification of Unknown Materials by
Gamma-Ray Spectroscopy., Howerton, R.J.;
Eggens, C.J., Lawrence Livermore Laboratory,
Livermore, CA, January 1980, FORTRAN:
LRLTRAN, CDC-7600, AVAIL: NTIS
- UCID-18749 ACTLMFE; GPC
Flux Weighted Group Cross Sections Based on the
ACTLMFE Data File., Howerton, R.J.,
Lawrence Livermore Laboratory, Livermore, CA,
July 1980, ASCII, AVAIL: NTIS and Doses Due
to the Inhalation and Ingestion of Radioactive
Aerosols., Sullivan, R.E., Office of Radiation
Programs, Washington, D.C., June 1977,
AVAIL: NTIS
- PNL-2970 GETOUT
GETOUT: A Computer Program for Predicting
Radionuclide Decay Chain Transport Through
Geologic Media., DeMier, W.V.; Cloninger, M.O.;
Burkholder, H.C.; Liddell, P.J., Battelle Pacific
Northwest Laboratory, Richland, WA, August
1979, AVAIL: NTIS
- PNL-3209 PABLM
PABLM: A Computer Program to Calculate
Accumulated Radiation Doses from Radionuclides
in the Environment., Napier, B.A.; Kennedy, Jr.,
W.E.; Soldat, J.K., Battelle Pacific Northwest
Laboratory, Richland, WA, March 1980,
AVAIL: NTIS
- SAND-79-1666 IONMIG
Prediction of the Migration of Several Radionuclides
in Ocean Sediment with the Computer Code
IONMIG: A Preliminary Report., Russo, A.J.,
Sandia National Laboratory, Albuquerque, NM,
May 1980, AVAIL: NTIS

- UCRL-15188 BIODOSE
User's Manual for Biosphere and Dose Simulation
Program (BIODOSE), Duffy, J.J.; Bogar,
G.P., Analytic Sciences Corporation, Reading,
MA, January 1980, AVAIL: NTIS
- UCRL-15208 BIODOSE
Sources of Data for Biodose Simulations Used in
Bedded Salt Repository Analysis., Duffy, J.J.,
Lawrence Livermore Laboratory, CA, January
1980, AVAIL: NTIS
- UCRL-84520; CONF-800731-2 AERIN
Use of AERIN Code for Determining Internal Doses
of Transuranic Isotopes., King, W.C.,
Lawrence Livermore Laboratory, CA, June
1980, AVAIL: NTIS

RADIATION SHIELDING INFORMATION CENTER

RSIC Periodic Distribution Query

The *RSIC Newsletter* carries information about RSIC products and services. Do you wish to continue to receive it? _____. If so, please fill out the form below as completely as possible and mail it **immediately**. We will purge from the distribution list the names of those not responding on or before **June 1**. Please print or type the name and mailing address. Use additional paper as necessary.

A. Name: _____
 Organization: _____
 Address: _____
 Telephone No: _____

(Commercial) (ETS)

B. Organization/institution type (Domestic __, Foreign __):
utility __, consultant __, industry-owned laboratory __, industrial
vendor __, architect-engineer __, government laboratory __, government
agency __, government contractor __, university __, health care organization __, other
(specify):

C. What are the research areas in which you are engaged?

Breeder Reactor	_____	Shipping Casks	_____
Light Water Reactor	_____	Fuel Cycle	_____
Fusion	_____	Health Physics	_____
Fusion - Fission Hybrid	_____	Occupational Exposure	_____
Weapons	_____	Radiation Damage	_____
Accelerators	_____	Environmental Exposure	_____
Space Shielding	_____	Activation and Heating	_____
Other			

D. Please indicate your source of financial support; if more than one sponsor, indicate proportionate fraction of time spent on each. **This information is essential.**

DOE - Breeder	____%	Defense Nuclear Agency	____%	NRC	____%
DOE - Fusion	____%	Army	____%	Utility	____%
DOE - Military	____%	Navy	____%	EPRI	____%
DOE - LWR	____%	Air Force	____%	State	____%
DOE - Other	____%	Defense - Other	____%	Private	____%

Other (Specify) _____

E. Have you placed your work in RSIC? _____. If yes, indicate below if it is time for an update. Do you have publications, computer codes, or data which you wish to contribute? Please comment:

PLEASE RETURN TO: *Radiation Shielding Information Center
Oak Ridge National Laboratory
P. O. Box X
Oak Ridge, Tennessee 37830*