

# RSIC Newsletter



RADIATION SHIELDING INFORMATION CENTER

## OAK RIDGE NATIONAL LABORATORY

OPERATED BY UNION CARBIDE CORPORATION • FOR THE U.S. ENERGY RESEARCH AND DEVELOPMENT ADMINISTRATION

POST OFFICE BOX X •  
OAK RIDGE, TENNESSEE 37830

No. 137

May 1976

*Advice comes too late when a thing is done.*

*...Anonymous.*

### SPECIAL SESSIONS OF ANS SHIELDING AND DOSIMETRY DIVISION AT TORONTO

Special sessions at the Toronto ANS meeting June 13-18 of interest to shielders include: *Environmental Radiological Monitoring* (Monday a.m.), *Radiation Environments in Nuclear Reactor Power Plants* (Monday p.m.), *Fusion Reactor Nuclear Analysis Methods and Codes* (Tuesday a.m.), *Radiation Streaming* (Wednesday a.m. and p.m.), and *Radiation Shielding for Fusion Reactors* (Thursday a.m.). The Canadian Nuclear Association program includes a session on *Control of Reactor Radiation Fields* on Wednesday at 3:30 p.m.

### CANADIAN PARTICIPATION IN ANS SOUGHT

In connection with the forthcoming joint meeting of the American Nuclear Society and the Canadian Nuclear Association, we have been asked to help find ways to encourage increased Canadian-U.S. cooperation through participation in the ANS Shielding and Dosimetry (soon Radiation Protection and Shielding) Division. Probably the business meeting of the Division, to be held after the Wednesday afternoon special session (Wentworth Room), affords the best opportunity for discussions of this matter. We encourage all those interested to make an effort to attend.

### NEWS ON THE FIFTH INTERNATIONAL CONFERENCE ON REACTOR SHIELDING

The Call for Papers for the Fifth International Conference on Reactor Shielding to be held at the Hyatt Regency Hotel in Knoxville, Tennessee, April 18-22, 1977, has appeared in the April issues of the RSIC Newsletter and Nuclear News (p. 148). Deadline for summaries (4 copies in English, 300 words, no figures): postmarked August 20, 1976. Mail summaries to: S.A.W. Gerstl, Program Committee Chairman, Los Alamos Scientific Laboratory, T-1, MS-269, P. O. Box 1663, Los Alamos, NM 87545 USA.

The General Chairman, David Trubey, has announced the appointment of Gary A. Vivian, Ontario Hydro, to the Technical Program Committee. He has also announced that commercial exhibits will be a part of the conference. Robert T. Santoro of ORNL has been appointed Exhibits Chairman.

With IAEA support, partial financial assistance to scientists from developing countries is expected to be available. For further information, contact D. K. Trubey, General Chairman, RSIC, ORNL, P. O. Box X, Oak Ridge, TN 37830.

### UNFOLDING SEMINAR WORKSHOP IN APRIL

The two-day seminar-workshop on Radiation Energy Spectra Unfolding was successfully concluded on April 13 with about 60 persons in attendance. There were 20 papers presented in the seminar portion. We gratefully thank the leaders of the workshops: Rich Johnson (Purdue University) and Bernie Wehring (University of Illinois at Urbana-Champaign) who led the session on the FORIST code, Francis Kam (ORNL), leader of the workshop on activation detectors, and Walter Meyer (University of Missouri at Columbia) who led discussions on proposed benchmark problem efforts. The proceedings will be issued as ORNL-RSIC-40.

### EUROPEAN SHIELDING INFORMATION SERVICE

#### *SEMINAR WORKSHOP ON THE MONTE CARLO CODE TRIPOLI, Ispra, October 18-19, 1976*

The seminar-workshop on the French Monte Carlo multigroup TRIPOLI code, announced in the ESIS Newsletter No. 16, and April RSIC Newsletter, will be organized at Ispra the 18-19 October 1976.

IF YOU CHANGE YOUR ADDRESS, please notify us (including Building and Room No. where needed). *Third Class Mail* is returned to us at our expense if the addressee has moved. If your mail is returned, your name will be deleted from our distributions until we hear from you.

A short description of the TRIPOLI code is given in the ESIS Newsletter No. 16. The seminar-workshop will have the following program.

### PROGRAM

#### October 18, 2 p.m.

1. Opening of the seminar-workshop
2. Presentation of the code
3. Problems in the distribution and implementation of the code

#### October 19, 9 a.m.

4. A simple test case
5. A complex test case
  - a) Description of the problem
  - b) Input data

*Afternoon session starting at 2 p.m.*

- c) Output of the complex case
6. General discussion on TRIPOLI and other Monte Carlo codes for shielding calculations.

Official languages English and French; simultaneous translation will be provided.

TRIPOLI treats by the Monte Carlo method the slowing down and diffusion of neutrons in 3D source problems. The version of the code which is distributed at the present is essentially oriented towards the solution of reactor shielding problems. The code is written for IBM computers and uses about 400 K bytes of storage. The geometry of the problem may be described by any set of VOLUMES, each of them being homogeneous and limited by portions of first and second order surfaces. The program treats geometries which are repetitive by symmetry, translation, or rotation. The program enables the source distribution to be described in a very general way, either for the space or the energy or the angle dependence. Nuclear constants are taken from library tapes (LINDA), containing energy-point tabulations from different evaluations. At present LINDA includes data from UKNDL-73, from UNC (GENDA), as well as thermal data from the program LEAP, and response functions for many detectors (activation, damage, heating, and dose equivalent). The code treats absorption and elastic scattering, however anisotropic, while  $(n,n')$  and  $(n,2n)$  reactions are treated as isotropic in the center of mass system. Only one thermal group is available at the present. The number of energy points may be very high; transport of neutrons being simulated by energy intervals (MACROGROUPS), only the constants of the Macrogroup that is being treated need to be present in the fast storage. TRIPOLI calculates and prints energy spectra and responses averaged over volumes or surfaces; standard deviation of the computed quantities is also provided.

Those interested should notify the organizers at the following address:

ESIS  
EURATOM JRC  
21020-Ispra (Va)  
ITALY

### CALL FOR PAPERS

#### Symposium on Vulnerability and Survivability of Aerial and Surface Targets

The Ballistics and Vulnerability Division of ADPA has scheduled a symposium on Target Vulnerability and Survivability, to be held on 26-28 October 1976 at the Naval Surface Weapons Center, White Oak Laboratory, Silver Spring, Maryland.

Papers are being solicited in the areas of Methodology and Aerial, Land and Sea Targets. Vulnerability and Survivability aspects related to conventional, laser, and nuclear weapons will be considered.

Proposed titles and abstracts (200-300 words) should be submitted in duplicate no later than 15 June 1976. One copy should be submitted to Headquarters ADPA, Attn: Colonel P. H. Scordas, 740 15th Street, N. W., Washington, D.C. 20005, and the other copy to: Dr. William W. Murray, Associate Technical Director for Structural Mechanics, David W. Taylor Naval Ship Research and Development Center, Bethesda, MD 20007, Attn. Code No. 8 17.

Papers will be selected from the abstracts by the Executive Board, Ballistics and Vulnerability Division, for the various sessions and all authors submitting abstracts will be advised regarding selection early in July.

In addition at the end of each session it is planned that an invited Review Paper will be presented summarizing the progress and gaps in Methodology and Technology.

The security level of the symposium will be "SECRET NOFORN". Proposed paper titles and abstracts should be unclassified if possible, but may be classified if necessary to permit a better judgment of the content of the proposed paper.

#### **STATUS OF ORNL-RSIC-13, Volume IV**

Problems have been encountered and a delay has resulted in the publication of ORNL-RSIC-13, Vol. IV, "Abstracts of Digital Computer Code Packages Assembled by Radiation Shielding Information Center." Pending requests will be filled as soon as the first copies are off the press.

#### **PERSONAL ITEMS**

The U.S. Energy Research and Development Administration has announced the appointment of **William H. Hannum**, previously assistant director for reactor safety at ERDA headquarters, as deputy manager of the agency's Idaho Operations Office, which administers the Idaho National Engineering Laboratory.

**Clayton D. Zerby**, formerly technical services manager for Union Carbide Corporation's Nuclear Division's production facilities in Oak Ridge, Tennessee, has been named to head a newly established Office of Waste Isolation in the Division. The new office has been established as part of the ERDA waste management program.

#### **VISITORS TO RSIC**

The following persons visited RSIC during the month of April: Paul W. Levy, Brookhaven National Laboratory, Upton, New York; E. Wachspress, General Electric, KAPL, Schenectady, New York; Farwell Smith, Energy Research and Development Administration, Washington, D.C.; Richard Shimko, Combustion Engineering, Windsor, Connecticut; Terry Stupar, Cincinnati General Hospital, Cincinnati, Ohio; Charles Peacock, NASA Marshall Space Flight Center, Huntsville, Alabama; T.M.A. Holst, and E. H. Brehm, Brown, Boveri & Cie, Mannheim, Germany; Abbas M. Marafie, University of Liverpool and Kuwait; Gunther Dietze, Physikalisches-Technische Bundesanstalt, 33 Braunschweig, West Germany; H. Alan Robitaille, Defense Research Establishment, Ottawa, Canada; Janet Luhmann, Aerospace Corporation, Los Angeles, CA; Helga Gerstner, ORNL Information Center Complex and W. Y. Yoon, Oak Ridge National Laboratory, Oak Ridge, TN.

#### **CHANGES TO THE DATA LIBRARY COLLECTION**

##### **DLC-27/AMPX01(104,22)**

The 104 neutron, 22 gamma-ray group cross section library has been updated to correct an error in the gamma-ray group transfer matrices. The error was pointed out by **H. Alan Robitaille** of the Defence Research Establishment Ottawa, Ottawa, Ontario, Canada. The error was introduced when DLC-27 was updated in September 1974, so users of DLC-27B will encounter erroneous results if they are looking at

problems involving gamma-ray transport (primary or secondary). The corrected library is designated DLC-27C. A reel of magnetic tape should accompany requests for the updated library.

**BOOK REVIEW: ENGINEERING COMPENDIUM ON RADIATION SHIELDING, VOL. II SHIELDING MATERIALS, Springer Verlag (New York, Heidelberg, Berlin) 1975**

It has been said that the possession of materials, the understanding of materials and the ability to use materials have been the determinants of every civilization the earth has ever known. If this be granted, then *information* on materials is an even more basic building block in a flourishing society. We need to know the amounts and the qualities of the material resources we possess (or lack). We must record, catalog and retrieve the myriad facts, theories and observations that constitute our understanding of materials, and we must have sufficient handbooks, manuals, texts and tutorial works to guide our citizens in their use. Despite this basic and critical position of materials information, we have become all too familiar with what may be called the materials information syndrome among the users of this information. This syndrome comprises bewilderment, apprehension, dismay, frustration, and outrage: *Bewilderment* with respect to the enormous volume and diversity of needed sources of information; *Apprehension* as to the quality and reliability of those facts; *Dismay* at the redundancy, gaps, and lack of coordination between information sources and systems; *Frustration* with the mechanics of search, retrieval and manipulation of information from the general store; and *Outrage* at the cost of seeking and locating needed information.

...J. H. Westbrook, General Electric Company, in "Requirements for Fulfilling a National Materials Policy," Proceedings of a Conference by the Federation of Materials Societies for the Office of Technology Assessment, United States Congress, August 11-16, 1974, p. 31.

Not long ago this reviewer attempted to assist Norman Schaeffer when he was preparing his popular text, *Reactor Shielding for Nuclear Engineers\**. It was axiomatic to us that no text on shielding would be complete without general information on shielding materials. After all—what is shielding but properly selected, arranged, and possibly shaped material? The fact that the resulting section on material comprises less than 3 percent of the book demonstrates not the relative importance of relevant materials information but the critical scarcity of it!

Now there is available at long last the final volume of the IAEA-sponsored **Engineering Compendium on Radiation Shielding**: Vol. II, **Shielding Materials**. Like previously published volumes (Vol. I, **Shielding Fundamentals and Methods** and Vol. III, **Shield Design and Engineering**, this text is a blending of contributions of many specialists (22) organized by the editor Anost Homig of Brno Technical University, with assistance of H. E. (Gene) Hungerford of Purdue University—both highly experienced in practical shield design.

This treatise covers in 436 well-indexed pages the nuclear, physical, and mechanical properties of substances commonly employed specifically as shields and also a few materials likely to be found in or near reactor cores and which contribute to the attenuation (e.g., uranium, copper, aluminum, beryllium, and graphite). Concrete—including the heavy and water-bearing concretes—occupies many pages. The book is

divided into the following major sections: gamma-ray shield materials (including water, transparent materials, and concretes), neutron-gamma-ray shields (including lead, borated materials, concretes, and air), laminated shields (including construction and testing), effects of heating on properties of Concrete, and finally the optimization of material choice and thicknesses. An addendum discusses iron mortars which can be useful in special applications.

Is the book useful? This reviewer, in the short time he has had a copy, found reason to seek radiation damage data for styrene-butadiene rubber and for cost data on concrete shields. He found it.

Aside from the cost (\$137.60), the greatest defect is similar to those of the earlier volumes—the long period of preparation results in a lack of recent work. For material properties, however, we would judge that this is not as serious as for the transport analysis methods treated elsewhere. All in all, we conclude that this book is a must for any shield design group. Like the other volumes, it is beautifully printed and bound.

...D. K. Trubey

\*Available from NTIS as TID-25951, \$13.60.

THIS LITERATURE IS ON ORDER. IT IS NOT IN OUR SYSTEM. PLEASE ORDER FROM NTIS OR OTHER AVAILABLE SOURCE AS INDICATED.

**REACTOR AND WEAPONS RADIATION  
SHIELDING LITERATURE**

AAEC/E-359

Neutron Yields and Energy Spectra from the  
Thick Target Li(p,n) Source.

Ritchie, A.I.M.

June 1975

Dep., NTIS (U.S. Sales Only) \$3.50

BNL-20713

Thermal Design and Analysis of Steady State  
Graphite Blankets.

Fillo, J.A.; Powell, J.R.

1975

NTIS

CNEA-393 (In Spanish)

Neutron Thermalization in Light Water.  
Spectrum Measurements and Calculations.

Abbate, M.J.; Lolich, J.V.

1975

Dep., NTIS (U.S. Sales Only) \$4.00

CONF-751101-29

Method for Processing ENDF/B Photon Form  
Factor Data.

Lucius, J.L.; Greene, N.M.

1975

Dep., NTIS \$4.50

CONF-751125-75

Design of an EPR Blanket.

Bettis, E.S.; Huxford, T.J.; McAlles, D.G.;

Santoro, R.T.; Watts, H.L.; Williams, M.L.

1975

NTIS

**Czechoslovak Patent 151677 Int. Cl. G21c15/16 (In  
Czech)**

**Liquid-Metal Steam Generator.**

Dubsek, D.

February 22, 1973

EPRI-221 (Technical Report)

Development of ENDF/B-IV Multigroup  
Neutron Cross-Section Libraries for the LEOPARD  
and LASER Codes.

Jenquin, U.P.; Stewart, K.B.; Heeb, C.M.

July 1975

Electric Power Research Institute, 3412 Hillview  
Avenue, Palo Alto, California 94304

ERDA-76-34

Fusion Power Research and Development.  
Summary Report on Magnetic Confinement  
Systems.

Division of Controlled Thermonuclear Research,

ERDA

January 1976

ERDA-tr-66

Selected USSR Papers on Plasma Physics and  
Controlled Thermo-Nuclear Fusion.

Division of Controlled Thermonuclear Research,

ERDA

October 1975

NTIS \$5.45

ERDA-tr-66, pp.19-34

Developments Related to Construction of  
Tokamak-Based Thermonuclear Reactors.

Golovin, I.N.

October 1975

NTIS

- EUR-5339  
*A Method to Solve the Integral Transport Equation Employing a Spatial Legendre Expansion.*  
 Hartman, J.  
 June 1975  
 NTIS (U.S. Sales Only)
- EURFNR-1243; KFK-2074 (In German)  
 Monte Carlo Calculation of Quotients in Reactor Physics.  
 Brandl, V.  
 February 1975  
 Dep., NTIS
- HEDL-SA-948; CONF-751026-22  
 Application of Damage Functions to CTR Component Fluence Limit Predictions.  
 Simons, R.L.; Doran, D.G.  
 1975  
 Dep., NTIS \$4.25
- IEA-38  
 Health Physics.  
 Poston, J.W.  
 December 1974  
 Dep., NTIS (U.S. Sales Only) \$9.00
- KFK-2046, pp.1-32  
 Resonance Capture Measurements on Structural Materials with Large Liquid Scintillators.  
 Froehner, F.H.  
 In: *Neutron Capture in the keV Energy Range in Structural Materials for Fast Reactors. Proceedings from NEACRP/NEANDC Specialist Meeting; Karlsruhe, F. R. Germany (8 May 1973).*  
 May 1975  
 NTIS
- KFK-2046, pp.51-62  
 Absolute Capture Cross Sections Measurements of Au, Cr, Fe, Ni Between 70 keV to 550 keV.  
 Le Rigoleur, C.; Arnaud, A.; Taste, J.  
 May 1975  
 NTIS
- KFK-2046, pp.63-72  
 Neutron Capture in Reactor Structural Materials.  
 Macklin, R.L.  
 May 1975  
 NTIS
- KFK-2046, pp.73-78  
 Some Problem Areas in Capture Cross-Section Measurements.  
 Moxon, M.C.; Gayther, D.B.; Sowerby, M.G.  
 May 1975  
 NTIS
- KFK-2046, pp.79-92  
 Capture Cross Section Measurements on Reactor Structural Materials with a Large Liquid Scintillator Detector.  
 Spencer, R.R.; Beer, H.  
 May 1975  
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- KFK-2046, pp.93-97  
 Summary on Topic I: Experimental Data.  
 Froehner, F.H.  
 May 1975  
 NTIS
- KFK-2046, pp.99-155  
 Comparison Between Single Level and Multilevel Calculations of Neutron Cross Sections in the Resonance Region.  
 Martinelli, T.; Menapace, E.; Motta, M.; Panini, G.C.  
 May 1975  
 NTIS
- KFK-2046, pp.156-189  
 Evaluation of the Neutron Cross-Sections of Natural Nickel and Its Stable Isotopes Below 600 keV.  
 Moxon, M.C.  
 May 1975  
 NTIS
- KFK-2046, pp.190-221  
 Thermal Cross Sections and Resonance Parameters Recommended in the new BNL 325 Vol.I for Cr, Fe, and Ni.  
 Pearlstein, S.  
 May 1975  
 NTIS
- KFK-2046, pp.251-256  
 Summary on Topic II: Evaluated Data.  
 Ribon, P.  
 May 1975  
 NTIS

- KFK-2046, pp.317-342  
 Comparison of Effective Capture Cross Sections and Doppler Coefficients for Structural Materials Calculated by Three Evaluated Nuclear Data Files.  
 Takano, H.; Ishiguro, Y.  
 May 1975  
 NTIS
- LA-6145-MS  
 Coarse-Mesh Rebalance Methods Compatible with the Spherical Harmonic Fictitious Source in Neutron Transport Calculations.  
 Miller, W.F., Jr.  
 October 1975  
 Dep., NTIS \$4.00
- LA-6209-SR  
 Fission Product Decay Heat Studies as of December 15, 1975.  
 Yarnell, Y.L.; Bendt, P.J.  
 January 1976  
 NTIS
- LA-6219  
 LINX and BINX: CCCC Utility Codes for the MINX Multigroup Processing Code.  
 MacFarlane, R.E.; Kidman, R.B.  
 February 1976  
 NTIS
- LA-UR-75-1247  
 LTPF: A Linear Theta-Pinch Neutron Source.  
 Ellis, W.R.  
 1975  
 NTIS
- LA-UR-75-1284; CONF-750631-4  
 Some Topics Concerning N-N and N-D Experiments at Medium Energy.  
 Simmons, J.E.  
 1975  
 NTIS
- ORNL-4992  
 A Methodology for Calculating Radiation Doses from Radioactivity Released to the Environment.  
 Killough, G.G.; McKay, L.R. (Comps.)  
 March 1976  
 NTIS \$12.50
- ORNL-TM-3706  
 AMPX: A Modular Code System for Generating Coupled Multigroup Neutron-Gamma Libraries from ENDF/B.  
 Greene, N.M.; Lucius, J.L.; Petrie, L.M.; Ford, W.E., III; White, J.E.; Wright, R.Q.  
 March 1976  
 NTIS \$10.75
- ORNL/TM-4960 (Applied Technology)  
 Analysis of the TSF First-Fission Stored-Fuel Experiment for the Fast Test Reactor.  
 Childs, R.L.; Mynatt, F.R.; Abbott, L.S.  
 March 1976  
 ERDA, TIC, P.O. Box 62, Oak Ridge, Tenn. 37830 \$4.00
- ORNL/TM-5057 (Applied Technology)  
 Analyses of the TSF First-Fission Stored-Fuel and Ex-Vessel Low-Level Flux Monitor Experiments for the Clinch River Breeder Reactor.  
 Childs, R.L.; Mynatt, F.R.; Abbott, L.S.  
 March 1976  
 ERDA, TIC, P.O. Box 62, Oak Ridge, Tenn. 37830 \$4.00
- ORNL-TM-5072  
 Measurement of Secondary Neutrons and Gamma Rays Produced by Neutron Interactions in Aluminum over the Incident Energy Range 1 to 20 MeV.  
 Morgan, G.L.  
 November 1975  
 Dep., NTIS \$4.00
- ORNL-TM-5126  
 Application of ALAP Concept to Exposure of Workers at Light-Water Reactors.  
 Dickson, H.W.; Cottrell, W.D.; Jacobs, D.G.  
 November 1975  
 Dep., NTIS \$5.45
- ORNL/TM-5293; Thesis  
 Mathematical Descriptions of a One- and Five-Year Old Child for Use in Dosimetry Calculations (Thesis).  
 Hwang, J.M.L.; Shoup, R.L.; Warner, G.G.; Poston, J.W.  
 March 1976  
 NTIS
- ORNL/TM-5295  
 The AXMIX Program for Cross Section Mixing and Library Arrangement.  
 Haynes, G.C.  
 March 1976  
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- ORNL/TM-5296  
AMP (Activity Manipulation Program).  
Engle, W.W., Jr.  
March 1976  
NTIS
- ORNL/TM-5337  
Radiation Insult to the Active Bone Marrow as  
Predicted by a Method of CHORDS.  
Jones, T.D.  
March 1976  
NTIS \$4.00
- ORNL/TM-5338 (Applied Technology)  
Analyses of the Preliminary In-Vessel and  
Enclosure Shield System Designs for the Clinch  
River Breeder Reactor (July, 1973 - July, 1975).  
Engle, W.W., Jr.; Williams, L.R.; Swanks, J.H.;  
Mynatt, F.R.; Abbott, L.S.  
March 1976  
ERDA, TIC, P.O. Box 62, Oak Ridge, Tenn.  
37830 \$5.00
- ORNL/NSF/EATC-12  
Analytical Analysis of Soil-Moisture and  
Trace-Contaminant Transport.  
Larson, N.M.; Reeves, M.  
March 1976
- R.M.G. Note 75/19  
Review of Activation Methods for the  
Determination of Neutron Flux Density Spectra.  
Zijp, W.L.  
September 1975  
Reactor Centrum Nederland
- RCN-232  
Neutron Spectra in the STEK Facility,  
Determined with the SAND-II Activation  
Technique.  
Zijp, W.L.; Nolthenius, H.J.; Baard, J.H.  
September 1975  
NTIS
- RCN-75-115 (Restricted Distribution)  
Comparison of SAND-II and RFSP-JUL  
Spectrum Unfolding Codes for Several Neutron  
Spectra in the STEK Facility.  
Fisher, A.; Nolthenius, H.J.; Zijp, W.L.  
September 1975  
Reactor Centrum Nederland
- RCN-75-131 (Restricted Distribution)  
Review of Activation Methods for the  
Determination of Neutron Flux Density Spectra.  
Zijp, W.L.  
October 1975  
Reactor Centrum Nederland
- RCN-76-019 (Restricted Distribution)  
Cross Section Library CESNEF (presented in the  
SAND-II format).  
Kramer, L.G.A.; Nolthenius, H.J.  
February 1976  
Reactor Centrum Nederland
- RCN-76-020 (Restricted Distribution)  
Cross Section Library LAPENAS (presented in  
the SAND-II format).  
Kramer, L.G.A.; Nolthenius, H.J.; Zijp, W.L.  
January 1976  
Reactor Centrum Nederland
- RP-8A  
Radiation Shielding Design and Analysis  
Approach for Light Water Reactor Power Plants.  
Stone and Webster Engineering Corp., Boston,  
Mass.  
May 1975  
Stone and Webster Engineering Corp., Boston,  
Mass.
- STI/PUB-381(Vol.2), pp.251-273; CONF-741105-P2,  
pp.251-273  
Radiation Damage by Neutrons to Materials in  
DT Fusion Reactors.  
Kulcinski, G.L.  
1974  
IAEA, Vienna
- UCRL-50400, Vol.5, Rev.1  
An Integrated System for Production of Neutronics  
and Photonics Computational Constants. Volume 5,  
Revision 1. CLYDE, A Code for the Production of  
Computational Constants from Nuclear Data.  
Doyas, R.J.; Dye, R.E.; Howerton, R.J.; Perkins,  
S.T.  
September 30, 1975  
NTIS
- UCRL-50400, Volume 15, Part A  
The LLL Evaluated Nuclear Data Library  
(ENDL): Evaluation Techniques, Reaction Index,  
and Descriptions of Individual Evaluations. An  
Integrated System for Production of Neutronics and  
Photonics Computational Constants.  
Howerton, R.J.; Cullen, D.E.; Haight, R.C.;  
MacGregor, M.H.; Perkins, S.T.; Plechaty, E.F.



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- UCRL-51969  
Computerized Data Base of the Fundamental Constants of Nature.  
Henry, E.A.; Hampel, V.E.  
December 3, 1975  
Dep., NTIS \$4.00
- UCRL-77465  
Controlled Fusion Physics: Experimental.  
Post, R.F.  
October 23, 1975  
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- WARD-2171-54 (Applied Technology)  
FFTF Shielding Design and Analysis Summary Report.  
Westinghouse Electric Corporation  
June 1975  
ERDA, TIC, P.O. Box 62, Oak Ridge, Tenn. 37830 \$21.25
- Atomkernenergie, 26(2), 76-79  
Measurement of the Energy Spectrum of Prompt Neutrons from the Fission of Pu-239 by 0.215 MeV Neutrons.  
Knitter, H.H.  
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Monte Carlo Calculations of Energy Response for Low-Energy Gamma Rays in Sodium-Iodide Crystals.  
Mukoyama, T.  
March 1975
- Computer Physics Communications, 10(5), 282-291  
Bicubic Spline Function Approximation of Solution of Fast-Neutron Transport-Equation.  
Chamayou, J.M.  
1975
- Energetika, 6, 270-274 (In Czech)  
Radiation-Resistant Bituminous Materials.  
Reiter, R.  
June 1975
- Fizika, 5, 37-42  
Prompt Gamma-Ray Spectra from the Radiative Capture of 14.1 MeV Neutrons in Cu, Se, Br, In, and I.  
Budnar, M.; Cvelbar, F.; Ivkovic, V.; Perdan, A.; Potokar, M.  
1973
- Fizika, 7, 157-161  
Angular Distribution of Neutron Capture Gamma-Rays in the Semidirect Capture Model.  
Likar, A.; Potokar, M.; Cvelbar, F.  
1975
- J. Inorg. Nucl. Chem., 37(11), 2243-2246  
Distribution of Fission Yields in the 3.0 MeV Neutron-Induced Fission of <sup>238</sup>U.  
Harvey, J.T.; Adams, D.E.; James, W.D.; Beck, J.N.; Meason, J.L.; Kuroda, P.K.  
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- J. Math. Phys., 16(9), 1840-1843  
A Three-Dimensional Neutron Transport Problem.  
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- J. Math. Phys., 16(10), 2166-2171  
Positive Solution of a Time and Energy Dependent Neutron Transport Problem.  
Pao, C.V.  
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- J. Nucl. Sci. Technol.(Tokyo), 12(6), 384-387  
A Numerical Method for Calculation of Time-Dependent Neutron Transport Equation.  
Otomo, N.; Inoue, K.  
June 1975
- Kenchiku Kenkyusho Nenpo, 1973, 141-150 (In Japanese)  
Radiation Shielding Design for Irradiation Facilities.  
Ito, K.  
December 1970
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Neutron Angular Distribution and Energy Spectra for Various Space Directions in the Vicinity of Concrete Shields.  
Doerschel, B.  
1975
- Kerntechnik, 17(8), 364-370  
Practical Handling of the Finite Element Method. Pt.2.  
Stelzer, F.  
August 1975

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