

RADIATION SHIELDING INFORMATION CENTER

OAK RIDGE NATIONAL LABORATORY

OPERATED BY UNION CARBIDE CORPORATION + FOR THE U.S. ATOMIC ENERGY COMMISSION

POST OFFICE BOX X • OAK RIDGE, TENNESSEE 37831

No. 117

August 1974

The degree to which we have eliminated secrecy is the measure of our civilization.

...Frank P. Zeidler

DATA LIBRARY COLLECTION CHANGES

Changes were made to the DNA Working Cross Section Library as follows:

The fluorine and tantalum evaluations have been modified recently, the updated versions being designated as DNA MAT 4509 MOD 1 fluorine and DNA MAT 4179 MOD 4 tantalum-181. The changes are summarized below.

Fluorine MAT 4509 ORNL

MOD 1 July 1974

A complete evaluation including neutron and gamma-ray production data replaces MOD 0, which has only gamma-ray production data. Changes were made to the gamma-ray production files as well, with data being added for the gamma-rays from the (n,α) reaction and replaced for the 197 and 110 keV (n,n^{-1}) levels. For both reactions, the gamma-ray production data were based on calculations of the corresponding neutron cross sections.

Tantalum-181 MAT 4179 LLL

MOD 4 July 1974

The partial inelastic cross sections were found to be in error in the energy range from 0.3 to 20 MeV. The problem was corrected by reducing the partial inelastic cross sections for MT=52-57 and 91, as well as total inelastic and total cross sections.

Changes were made to the DNA Processed Cross Section Library as follows.

DLC-31/(DPL-401/NEDT1) Neutron Element Data Tape in SAM-CE format. Data for Li-7, B-10, B-11, Mg, K, W-183, W-184, U-235, Pu-239, and Pu-240 were added in July 1974. One reel of magnetic tape required to obtain these added data (15,244 records). A total of three tapes to obtain the entire library (53985 records) written 7 track, 556 bpi, or one only, written 9 track, 800 bpi. DLC-31/(DPL-402/GPDT1)

Gamma-Ray Production Data Tape in SAM-CE Format. Data for Li-7, B-10, C-12, Mg, Fe, U-235, and U-238 added in July 1974. One reel of tape required to obtain entire library (10740 records).

A new data set was packaged during the month.

DLC-32/GAMTAB

Radioactive Decay Gamma-Ray Compilation ordered by ascending gamma-ray energy and by nuclide, in ascending mass number. References are also listed. Information includes gammaray energy and error, nuclide identification and abundance, half-life, and production mode. The package contains the source data used to generate the "Radioactive Decay Gammas" tables published in <u>Nuclear Data Tables</u>, Vol. 13, Nos. 2 and 3, 1974. Contributor: Savannah River Laboratory, Aiken, S. C., through the Argonne Code Center. Required for transmittal of 26,871 records of information: one reel of magnetic tape (9T,800), or two reels (7T,556) written unblocked.

MORE ENDF/B-IV DATA RELEASED

Two additional ENDF/B-IV tapes have been released by the National Neutron Cross Section Center (NNCSC) at Brookhaven National Laboratory (BNL). The recently released tapes are numbered 406 and 408. The remainder of the ENDF/B-IV tapes are expected to be released during the summer. The contents of the tapes released thus far are shown in the accompanying table. Requests for the data should be directed to NNCSC at BNL, not to RSIC.

Contents of ENDF/B-IV Tapes Released as of August 1, 1974.

TAPE	MAT	MATERIAL	TAPE MAT	MATERIAL	TAPE	MAT	MATERIAL
401	1128	W-182	1172	Xe-128	403	1027	Sm-149
	1129	W-183	1173	Xe-129		1030	Gđ
	1130	W-184	1174	Xe-130		1083	Re-185
	1131	W-186	1175	Xe-131		1084	Re-187
	1169	Н-З	1176	Xe-132		1125	Rh-103
	1195	Ca	1177	Xe-133		1127	Ta-182
	1270	He-4	1178	Xe-136		1137	Tc-99
			1181	Kr-78		1138	Ag-107
402	1031	Dy-164	1182.	Kr-80		1139	Ag-109
	1032	Lu-175	1183	Kr-82		1141	Cs∽133
	1033	Lu-176	1184	Kr-83		1149	Cl
	1120	H-2	1185	Kr-84		1150	K
	1146	He-3	1186	Kr-86		1156	Na-23
	1170	Xe-124	1196	V		1160	B-11
	1171	Xe-126					

TAPE	MAT	MATERIAL	TAPE	MAT	MATERIAL	TAPE	MAT	MATERIAL
404	1043	U-234		1272	Li-7	406	1190	Ni
	1050	Pu-238		1273	B-10		1 <u>191</u>	Cr
	1056	Am-241		1289	Be-9		1192	Fe
	1057	Am-243		1294	Xe-135			
	1161	Pu-242		1296	Th-232	408	1274	C-12
	1162	Cm-244					1275	N-14
	1163	U-236	405	1193	Al-27		1276	0-16
	1269	Н		1194	Si		1288	РЬ
	1271	Li-6		1280	Mg			

CORRECTION TO PSR-51/SMUG

J. L. Lucius, Computer Sciences Division, Union Carbide Nuclear Division, Oak Ridge, and T. L. Yang, Ebasco, New York City, have called attention to an error in SMUG which occurs when the program is used to generate lower order expansion multigroup cross sections (P_0 or P_1). The error does not occur when cross section sets with more terms are produced. A correction suggested by Jim Lucius, and implemented in the RSIC package, is as follows:

In Subroutine MUG, add, as the first executable statement, T = 0.0.

SMUG was developed at ORNL to generate multigroup photon cross sections, and is designed to read ENDF formatted data.

SEMINAR-WORKSHOP ON CROSS-SECTION DATA GENERATION

In response to requests over a period of time, we are beginning to make plans for a Seminar-Workshop on cross-section data processing. It is time to reassess the state of the art and the Seminar-Workshop is probably an effective way to do it. We conclude that in a proposed cross-section data processing workshop we should include both point and multigroup data, and the entire neutron energy range from thermal data through fast through super-fast (>20 MeV) cross section data. It should include photon interaction and production processing as well.

We will appreciate having your comments, suggestions, and ideas. We will need the cooperation of the cross section code developers, the data processors, and the users of processed data in the planning, and invite your participation as a contributor and as an observer.

We are considering holding the meeting in early December in Oak Ridge. We invite your recommendations as to the most convenient date.

Watch the Newsletter for developments.

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CONFERENCES

The European Nuclear Society (ENS)/American Nuclear Society (ANS) have announced the First European Nuclear Conference to be held in the Paris Convention Center, Paris, France, April 21-25, 1975. Professionals interested in nuclear energy from all countries are urged to send contributed papers and to participate in this event which will be focused on the technological maturity achieved in the world and in Europe, with particular emphasis on the practical experience acquired from the implementation and exploitation of nuclear industrial facilities. The technical program of the Conference will consist of sessions formed from contributed and invited papers on topics of special current interest, such as Water Reactor, Gas Cooled Reactor and Fast Breeders.

Submission of summaries must reach the Executive Office by October 1, 1974. Information is available from Mr. Pierre Zaleski, ENC Executive Chairman, P. O. Box No. 27, 92140 Clamart, France.

A Call for Papers has been made on the following subjects: Design and Construction; Operating Experience; Fuel Performance; Fuel Management; Nuclear Safety and Protection; Quality Assurance and Reliability for Components and Fuel; Fuel Fabrication; Reprocessing, Transport and Waste; and Low- and High-Temperature Process Heat.

A Call for Papers has gone out for the Conference on Nuclear Cross Sections and Technology to be held at the Shoreham Americana Hotel, Washington, D.C., March 3-7, 1975. The purpose of the conference is to summarize the present status of Nuclear Cross Sections and Technology, to discuss future cross sections needs, and to provide opportunities for the exchange of information between nuclear scientists and engineers. Although contributions on all aspects of nuclear data are invited, the emphasis will be on the use of nuclear cross sections for applied purposes.

Original papers describing significant contributions in the following or related areas are invited: Applications of Nuclear Data to Fission Reactors; Applications of Nuclear Data to Fusion Reactors; Microscopic Data and Measurement Techniques; Standards; Benchmark Experiments and Sensitivity; Nuclear Data for Materials Analysis; Nuclear Material Safeguards and Management; Nuclear Data for Environmental Protection; Biomedical Applications of Nuclear Data; and Other Applications of Nuclear Data. The abstract must be sent by December 6, 1974, to Professor W. W. Havens, Jr., Division of Nuclear Science and Engineering, Columbia University, 520 West 120 Street, New York, N. Y. 10027.

Sponsors of the Conference are: American Nuclear Society, Reactor and Shielding Divisions; American Physical Society, Nuclear Physics Division; International Union of Pure and Applied Physics; National Bureau of Standards; and United States Atomic Energy Commission.

"Computational Methods in Nuclear Engineering" will be the title of a topical meeting planned for Charleston, S.C., April 15-17, 1975, sponsored by the Mathematics and Computation Division, and Savannah River Section of

ANS and the Savannah River Operations Office, USAEC. General chairman of the meeting is H. C. Honeck (SRL) and the technical program chairman is W. M. Stacey, Jr. (ANL, Ill.).

The meeting will feature innovative mathematical and computational methods for solving problems in nuclear engineering. Session topics include radiation transport, multidimensional static diffusion theory, multidimensional diffusion and transport theory dynamics, mechanics problems in reactors, application of optimal control theory to reactors, code systems and system software, environmental and economic modeling of nuclear plants, and a workshop on computer code exchange. All the sessions except the one on mechanics will present invited and contributed papers, and several will feature panel discussions.

The deadline for submitting abstracts for the International Symposium on Radiation Physics to be held November 30 to December 4, 1974 at the Bose Institute, Calcutta, India, has been set for September 15, 1974. They should be sent to Prof. A. M. Ghose, Head, Department of Physics, Bose Institute, Calcutta-700009, India. Papers may be contributed on the following topics: Interactions of gamma rays; neutron transport in bulk media; shielding of medium energy charged particle accelerators; and radiation dosimetry and instrumentation.

In addition to contributed papers, a number of invited review papers on subject matter of the Symposium, along with a few semi-technical lectures on related areas are proposed to be delivered during the sessions. Keynote addresses on various topics will be delivered by: Robert Hofstadter (Nobel Laureate), Stanford University, California; G. Drexler, Institute ür Strahlenschutz, Germany; Y. Furuta, Japan Atomic Energy Research Institute; A. M. Ghose, Bose Institute, Calcutta; T. Hyodo, Kyoto University, Japan; John H. Hubbell, Center for Radiation Research, National Bureau of Standards, Washington, D.C.; P.K. Iyenger, Bhabha Atomic Research Centre, Trombay, India; L. S. Kothari, Delhi University, Delhi, India; S. Makra, Central Research Institute for Physics, Hungarian Academy of Sciences, Budapest, Hungary; A. B. Smith, Argonne National Laboratory, Illinois; and D. K. Trubey, Radiation Shielding Information Center, Oak Ridge National Laboratory, Tennessee.

Abstracts of papers, preferably written in English, should be sent by September 15, 1974, indicating the subject category on the top left hand side of the abstract. The abstracts should be limited to 300 words. Complete papers should reach Calcutta by November 1, 1974.

VISITORS TO RSIC

Visitors to RSIC during the month of July were: S. J. Cipollo, Creighton University, Omaha, Neb.; J. Dagan, St. Luke's Hospital, New York, N.Y.; F. D. McDaniel, North Texas State University, Denton, Tex.; Y. Miura, University of California, Santa Barbara, Calif.; E. Schmidt; Euratom, Ispra, Italy; I. Tag (Egypt), at present with the Reactor Division, ORNL; J. Owings and J. Taylor, Computer Sciences Div., Union Carbide Nuclear Div. S. C. Roy, Bose Institute, Calcutta, India.

PERSONAL ITEMS

Captain Dean C. Kaul, USAF, former Radiation Transport Project Officer in the Radiation Directorate (STRA), Tactical Nuclear Division (RATN), of Headquarters, Defense Nuclear Agency (DNA) has retired from the Air Force. He has accepted a position as staff scientist in nuclear technical applications in the Chicago office of Science Applications, Inc. (SAI). His DNA responsibilities have been assumed by *Captain Ronald G. Powell*, USAF, who came to Headquarters from the Air Force Weapons Laboratory, Kirtland Air Force Base, New Mexico.

Marion L. Weiss, Breeder Reactor Operations of the General Electric Company, Sunnyvale, California, has accepted a guest assignment to the Oak Ridge National Laboratory to work for one year on shielding calculations in support of both experimental and detailed design of the breeder reactor.

A change of address has called to our attention that Art Reetz is now with NASA's Outer Planets Mission at Headquarters.

Linda Nickel, a student from Illinois College, Jacksonville, Illinois, is an RSIC summer participant under sponsorship of Oak Ridge Associated Universities (ORAU). She is working on the checkout of the BMT neutron moments code of C. R. Weisbin of Oak Ridge National Laboratory and P. D. Soran of Los Alamos Scientific Laboratory.

The following changes of address are noted: Dr. Michael Weinert, from Institut für Strahlenschutz, Stohl, to Munster, West Germany; Dr. Paul S. Pickard, from the University of Illinois to Reactor Study Division, Sandia Laboratories, Albuquerque, New Mexico.

RSIC welcomed a new staff member July 1 in the person of Ms. Ellen Bervier who will assist in the myriad details connected with secretarial work in an information analysis center.

JULY ACCESSION OF LITERATURE

The following literature cited has been ordered for review, and that selected as suitable will be placed in the RSIC Information Storage and Retrieval Information System (SARIS). This early announcement is made as a service to the shielding community. Copies of the literature are not distributed by RSIC. They may generally be obtained from the author or from a documentation center such as the National Technical Information Service (NTIS), Department of Commerce, Springfield, Virginia 22151.

RSIC maintains a microfiche file of the literature entered into SARIS, and duplicate copies of <u>out-of-print</u> reports may be available on request. Naturally, we cannot fill requests for literature which is copyrighted (such as books or journal articles) or whose distribution is restricted. THIS LITERATURE IS ON ORDER. It is NOT in our system. Please order from NTIS or other available source as indicated.

REACTOR AND WEAPONS SHIELDING

AFWL-TR-73-143: AD-766230-7 Time-Dependent Transport Via the Continued Fraction Approximation. Technical Report - January 1972-May 1973. Yingling, W.A. July,1973 NTIS \$4.50 BNL-18792; CONF-740402-37 Damage Energy Cross Section and Primary Recoil Spectra for CTR-Related Neutron Sources. Parkin, D.N.; Goland, A.N.; Berry, H.C. 1973 Dep., NTIS \$4.00 BN#L-SA-4968: CONF-740443-2 Consideration of Reactor Accident Exposure Guides for Plutonium. Bair, W.J. 1974 Dep., NTIS \$4.25 BRL-1665: AD-769711 In-and-Down Scattered Radiation in a Simple Concrete Structure. Final Report. Schmoke, M.A. September, 1973 NTIS \$4.75 BRL-1666: AD-769712 Sensitivity of the Transport of 14-MeV Neutrons to the Nitrogen - Elastic Angular Distribution. Beverly, W.B. September, 1973 NTIS \$3.75 CONF-690103-P1, pp.651-675 Gamma and Neutron Surface Dose Rate Measurements: Theoretical and Practical. In: Health Physics Operational Monitoring. Vol.1. Willis, C.A. (Ed.) Helgeson, G.L. 1972 Corder and Broach Science Publishers Inc. (1972) Górdon and Breach, Science Publishers, Inc. (1972), New York, N.Y. CONF-721045, pp.60-74; LA-5180-C, pp.60-74 Theoretical Aspects of Radiation Dosimetry. Caswell, R.S.: Berger, M.J. March, 1973 Dep., NTIS \$3.00 CONF-721045, pp.88-92; LA-5180-C, pp.88-92 Practical Dosimetry: High L.E.T. Radiation. Almond, P.R. March, 1973 Dep., NTIS \$3.00

```
CONF-721045, pp.93-96; LA-5180-C, pp.93-96

Radiation Quality and Its Biomedical Implications.

Bossi, H.H.

March, 1973

Dep., NTIS $3.00
CONF-721115, pp.27-31

Radiation Damage Units for Fast Reactor Steels.

Branman, J.I.; Etherington, E.W.; Nelson, R.S.;

Norgett, M.J.

1973

J. J. Tradiation Embrittlement and Green in Ruel
     In: Irradiation Embrittlement and Creep in Fuel Cladding
and Core Components. London, England, Nov. 9,1972.
British Nuclear Energy Society, London
CONF-721203, pp.45-57
Calculation and Comparison of Leakage Neutron Spectra.
Makra, S.: Palfalvi, J.; Zarand, P.
1973
             IAEA
CONF-721203, pp.141-176 (In Bussian)
Neutron Dose Distribution in Tissue-Equivalent Phantoms.
Obaturov, G.M.; Shalin, V.A.; Keirim-Markus, I.B.;
Kochetkov, O.A.
1973
             IAEA
CONF-721203, pp. 177-191
Standardizing the Fluence-to-Dose-Equivalent Conversion
Factors for Whole-Body Neutron Exposures.
Stevenson, G.R.; Hoefert, M.; Neufeld, J.; Rindi, A.;
Routti, J.T.; Pretre, S.
1973
             IAEA
CONF-721203, pp. 193-199
Experimental Determination of Dose-Equivalent and
Quality Factor in a Cylindrical Phantom Irradiated with
Reactor Neutrons.
             Pszona, S.
1973
             IAEA
CONF-721203, pp.211-222 (In Russian)
Estimating the Contribution of Neutrons Scattered Inside
Buildings to Total Radiation Doses.
Savinskii, A.K.: Pilyushkin, I.V.
1973
             IAEA
     Next Flight Estimation for the Fictitious Scattering
Monte Carlo Method.
Cramer, S.N.
1974
CONF-740608-6
             Dep., NTIS $4.00
```

HEDL-TME-74-7 Past Test Reactor Sodium Activity. Kidman, R.B.; Marr, D.R. January, 1974 AT, TIC(USAEC) IPP-1-143 Neutron Flux Distribution on the Wall of Toroidal Controlled Thermonuclear Reactor Devices with Elongated Cross Section. Becker, G.; Wunderlich, R. Pebruary, 1974 Dep., NTIS (U.S. Sales Only) \$4.25 IBI-133-73-17 Catalogue of Gamma-Ray and Isotope Data for Use in Neutron Activation Analysis. de Bruin, M.; Korthoven, P.J.M.; Faasse-van Peer, M.J.J.A. No Date Dep., NTIS (U.S. Sales Only) \$4.75 JPRS-60560, pp.208-211; CONF-720130, pp.208-211 Some Distinctive Features in Planning Protection Against the Radiation of Gas-Cooled (N204) Fast Reactors. Shchekin, Yu.K.; Guskina, L.N.; Tkachenko, S.E. November 16,1973 NTIS LA-5553-MS Estimated Values of Fractional Yields from Low Energy Fission and a Compilation of Measured Fractional Yields. Wolfsberg, K. May, 1974 NTIS LA-UR-74-592: CONF-740402-13 Some Neutronics Aspects of Laser-Fusion Reactors. Frank, T.; Dudziak, D.J.; Heck, E. 1974 Dep., NTIS \$4.00 LA-UR-74-600; CONF-740440-1 Application of Shielding Benchmark Experiments to the Testing of Nuclear Data, Calculational Methods, and Procedures. LaBauve, R.J.; Harris, D.R.; Muir, D.W.; Hennig, P.B. 1974 Dep., NTIS \$4.50 LA-UR-74-611; CONF-740402-30 Trasmutation and Atom Displacement Rates in a Reference Theta-Pinch Reactor. Dudziak, D.J. 1974 Dep., NTIS \$4.50 LA-UR-74-613; CONF-740402-33 Sensitivity of Neutron Multigroup Cross Sections to Thermal Broadening of the Fusion Peak. Muir, D.W. No Date Dop NTTS #4 00 Dep., NTIS \$4.00

LA-UR-74-629; CONF-740402-18 Comparative Analysis of D+T Fusion Reactor Radioactivity and Afterheat. Dudziak, D.J.; Krakowski, R.A. 1974 Dep., NTIS \$4.25 RT/FI-(74)-2 Nuclear Data and Integral Measurement Correlation for Past Reactors. Statistical Formulation and Analysis of Methods. The Consistent Approaca. Gandini, A.; Petilli, M.; Salvatores, M. 1974 Conitato Nazionale Energia Nucleare, Rome ORNL-RSIC-34, Vol.1 (Rev.) Defense Nuclear Agency Working Cross Section Library: Description and Contents. Roussin, R.W.; Wright, J.B. June, 1974 PSTC OPUT RSIC, ORNL ORN1-TM-4595 Approximate Numerical Solutions to the Time-Dependent Neutron Transport Equation Using the Task Algorithm. Tomlinson, E.T. July,1974 NTIS \$5.45 Review of Heavy-Ion Fusion Cross Section Results. Plasil, F. June, 1974 ORNL-TM-4599 NTIS SLA-73-159-A Recommended Use of SI Units (Metric System) at Sandia Laboratories. McDonald, G.C. July, 1973 Dep., NTIS \$3.25 UCRL-75627; CONF-740452-1 Calculations of a Fast Fission Blanket for DT Pusion Reactors with Two Evaluated Data Libraries. Haight, R.C.; Lee, J.D. April 11,1974 Dep., NTIS \$4.00 Y-1948 Validation of the "KENO" Code for Nuclear Criticality Safety Calculations of Moderated, Low-Enriched Uranium Systems. Handley, G.R.; Hopper, C.M. June 13, 1974 .

Geoexploration, 12(1), 1-10 Calculation of Yield Curves for Airborn Gamma-Ray Spectrometers. Holmberg, P.; Blomster, K.; Rieppo, R. 1974 Health Phys., 26(6), 483-488 Nuclear Power and Public Opinion. Kathren, R.L. June,1974 Health Phys., 27(1), 145-147 Gamma-Bay Attenuation Coefficients for Higher Green Plants. (Tech. Note) Tsoulfanidis, N. 1974 J. Belge Radiol., 56(5), 451-455 (In French) Integral Absorbed Dose. Its Interest in Radioprotection. Bercy, A. Kernenergie, 16(10), 317-321 (In German) Simple Methods for Calculating the Gamma-Ray Exposure Under a Radioactive Plume. Oppermann, R.; Krueger, F.W. October, 1973 Math. Biosci., 18 (3-4), 255-268 Invariant Imbedding and Radiation Dosimetry. VII. Finite Order Scattering and Transmission Functions of the Two Radiation Approximations in a Target Slab. Bellman, R.; Ueno, S.; Vasudevan, R. December, 1973 Nucl. Eng. Design, 27(2), 274-285 A Status Report on Computational Techniques for Finite Element Analyses. Melosh, R.J. May, 1974 Nucl. Instrum. Nethods, 113(4), 483-487 Subtraction of the Background and Automatic Peak Identification in Gamma-Ray Spectra Obtained from a Ge(Li) Detector. Volkov, N.G. December, 1973 Nucl. Instrum. Methods, 114(1), 113-119 A Monte Carlo Simulation of the Capture and Detection of Neutrons with Large Liquid Scintillators. Poitou, J.; Signarbieux, C. January, 1974 Nucl. Instrum. Methods, 114(1), 153-155 Transmission of Lead-Capture Gamma Rays with a Tin Absorber. Kawarasaki, Y. January,1974

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 Nucl. Instrum. Methods, 114(2), 205-231 proportional Counter Characteristics and Applications to Reactor Neutron Spectrometry. Verbinski, V.Y.; Giovannini, R. Januarr,1974 Nucl. Instrum. Methods, 116(3), 515-519 Internal-Source Method of Measuring Absolute Pair-Production Cross-Sections. Ayignome, F.T.; Blankenship, S.M. 1970 Nucl. Instrum. Methods, 117(2), 597-598 Photon Self-Absorption Effects - Potentially Large Error in Passive Gamma-Rays Assays. Porster, R.A.; Umbarger, C.J. 1971 Nucl. Phys. A, A223(2], 423-426 Differential Cross-Sections for Blastic-Scattering of Gamma-Rays for Energies from 145 keV to 750 keV. 386nd, F.; Schumacher, M. 1972 Nucl. Phys. A, A221(3), 573-592 6-11(a,t)H He Differential Cross Sections Between 0.1 and 1.6 HeV. Overley, J.C.; Sealock, R.N.; Shlers, D.H. March, 1974 Nucl. Sci. Eng., 54(3), 350-351 Nucl. Sci. Eng., 54(3), 235-253 Nucl. Sci. Eng., 54(3), 235-253 Nucl. Sci. Eng., 54(3), 235-253 Nucl. Sci. Eng., 54(3), 235-253 Nucl. Sci. Eng., 54(3), 254-262 Errorimental Equilibrium Angular Distribution. Johnson D. H. Nucl. Sci. Eng., 54(3), 254-262 Errorimental Rvidence of an Equilibrium Property in Gamma-Ray Angular Spectra Penetration Uti Not. Sci. Eng., 54(3), 254-262 Errorimental Evidence of an Equilibrium Property in Gamma-Ray Angular Spectra Penetrating Bulk Shields. Johnson D. H. July, 1974 Nucl. Sci. Eng., 54(3), 263-272 The Yield of Short-Lived Gamma-Ray Emitting Muclides from Fast- and Therma1-Meutron Fission. Area, F.P.; Dudey, M. D. Nucl. Sci. Eng., 54(3), 263-272 The Yield of Short-Lived Gamma-Ray Emitting Muclides from Fast- and Therma1-Meutron Fission. Area, R.P.; Dudey, N.D.; Gold, R. 	
 Pair-Production Cross-Sections. Avignome, F.T.; Blankenship, S.M. 1974 Nucl. Instrum. Methods, 117(2), 597-598 Photon Self-Absorption Effects - Poteatially Large Error in Passive Gama-Rays Assays. Porster, R.A.; Umbarger, C.J. 1974 Nucl. Phys. A. A223(2), 423-428 Differential Cross-Sections for Elastic-Scattering of Gama-Rays for Energies from 145 keV to 750 keV. Smend, F.; Schumacher, H. 1974 Nucl. Phys. A. A221(3), 573-592 6-Li(h,t)4 He Differential Cross Sections Between 0.1 and 1.8 HeV. Overley, J.C.; Sealock, R.M.; Ehlers, D.H. March, 1974 Nucl. Sci. Eng., 53(3), 350-351 Direct Calculation of the Time-Dependent Spatial Nosents of the One-Speed Neutron Transport Model. Gamarb, B-D. March, 1974 Nucl. Sci. Eng., 54(3), 235-253 Prediction of GamarBay Spectra at Deep Penetration Utilizing the Equilibrium Angular Distribution. Johnson, D.M. July, 1974 Nucl. Sci. Eng., 54(3), 254-262 Experimental Evidence of an Equilibrium Property in GamarBay Angular Spectra Penetrating Dulk Shields. July, 1974 	Proportional Counter Characteristics and Applications
 Photon Self-Absorption Effects - Potentially Large Error in Passive Gamma-Rays Assays. Porster, R.A.; Umbarger, C.J. 1974 Nucl. Phys. A, A223(2), 423-428 Differential Cross-Sections for Elastic-Scattering of Gamma-Rays for Energies from 145 keV to 750 keV. Smend, P.: Schumacher, M. 1974 Nucl. Phys. A, A221(3), 573-592 6-Li(0,t)4 He Differential Cross Sections Between 0.1 and 1.8 MeV. Overley, J.C.; Sealock, R.M.; Ehlers, D.H. March, 1974 Nucl. Sci. Eng., 53(3), 350-351 Direct Calculation of the Time-Dependent Spatial Noments of the One-Speed Neutron Transport Model. Ganapol, B.D. March, 1974 Nucl. Sci. Eng., 54(3), 235-253 Prediction of Gamma-Ray Spectra at Deep Penetration Utilizing the Equilibrium Angular Distribution. Johnson, D.M. July, 1974 Nucl. Sci. Eng., 54(3), 254-262 Experimental Evidence of an Equilibrium Property in Gamma-Ray Angular Spectra Penetrating Bulk Shields. Johnson, D.M.; July, 1974 	Pair-Production (togs-Sactions,
 Differential Cross-Sections for Elastic-Scattering of Gamma-Rays for Energies from 145 keV to 750 keV. Smend, F.; Schumacher, M. 1974 Nucl. Phys. A. A221(3), 573-592 6-Li (n,t)4 He Differential Cross Sections Between 0.1 and 1.8 MeV. Overley, J.C.; Sealock, R.M.; Ehlers, D.H. March, 1974 Nucl. Sci. Eng., 53(3), 350-351 Direct Calculation of the Time-Dependent Spatial Noments of the One-Speed Neutron Transport Model. Ganapol, B.D. March, 1974 Nucl. Sci. Eng., 54(3), 235-253 Prediction of Gamma-Ray Spectra at Deep Penetration Utilizing the Equilibrium Angular Distribution. Johnson, D.M. July, 1974 Nucl. Sci. Eng., 54(3), 254-262 Experimental Evidence of an Equilibrium Property in Gamma-Ray Angular Spectra Penetrating Bulk Shields. Johnson, D.M.; Woollam, P.B. July, 1974 	Photon Self-Absorption Effects - Potentially Large Error in Passive Ganna-Rays Assays. Forster, R.A.; Unbarger, C.J.
 and 1.8 MeV. Overley, J.C.; Sealock, R.M.; Ehlers, D.H. March, 1974 Nucl. Sci. Eng., 53(3), 350-351 Direct Calculation of the Time-Dependent Spatial Noments of the One-Speed Neutron Transport Model. Ganapol, B.D. March, 1974 Nucl. Sci. Eng., 54(3), 235-253 Prediction of Gamma-Ray Spectra at Deep Penetration Utilizing the Equilibrium Angular Distribution. Johnson, D.M. July, 1974 Nucl. Sci. Eng., 54(3), 254-262 Experimental Evidence of an Equilibrium Property in Gamma-Ray Angular Spectra Penetrating Bulk Shields. Johnson, D.M.; Woollam, P.B. July, 1974 	Differential Cross-Sections for Elastic-Scattering of Gamma-Rays for Energies from 145 keV to 750 keV. Smend, F.: Schumacher, M.
Direct Calculation of the Time-Dependent Spatial Moments of the One-Speed Neutron Transport Model. Ganapol, B.D. March, 1974 Nucl. Sci. Eng., 54(3), 235-253 Prediction of Gamma-Ray Spectra at Deep Penetration Utilizing the Equilibrium Angular Distribution. Johnson, D.M. July, 1974 Nucl. Sci. Eng., 54(3), 254-262 Experimental Evidence of an Equilibrium Property in Gamma-Ray Angular Spectra Penetrating Bulk Shields. Johnson, D.M.; July, 1974	and 7.8 NeV.
Nucl. Sci. Eng., 54(3), 254-262 Experimental Evidence of an Equilibrium Property in Gamma-Ray Angular Spectra Penetrating Bulk Shields. July, 1974	Nucl. Sci. Eng., 53(3), 350-351 Direct Calculation of the Time-Dependent Spatial Moments of the One-Speed Neutron Transport Model. Ganapol, B.D. March, 1974
Johnson, D.M.; Woollaw, P.B. July,1974	Johnson, D.N.
Nucl. Sci. Eng., 54(3), 263-272 The Yield of Short-Lived Gamma-Ray Emitting Nuclides from Fast- and Thermal-Neutron Fission. Larsen, R.P.; Dudey, N.D.; Heinrich, R.R.; Oldham, R.D.; Armani, R.J.; Popek, R.J.; Gold, R. July, 1974	Gamma-Kay Angular Spectra Penetrating Bulk Shields.
	Nucl. Sci. Eng., 54(3), 263-272 The Yield of Short-Lived Gamma-Ray Emitting Nuclides from Fast- and Thermal-Neutron Fission. Larsen, R.P.; Dudey, N.D.; Heinrich, R.R.; Oldham, R.D.; Armani, R.J.; Popek, R.J.; Gold, R. July, 1974

Nucl. Sci. Eng., 54(3), 273-285 Cobalt Fast-Neutron Cross Sections. Guenther, P.T.; Moldauer, P.A.; Smith, A.B.; Whalen, J.F. July, 1974 Nucl. Sci. Eng., 54(3), 300-316 Evaluation of Uranium-238 Neutron Cross Sections from Spectral Neasurements. Bluhm, H.; Gieg, G.; Werle, H. July,1974 Nucl. Sci. Enq., 54(3), 327-340 Anisotropic Neutron Diffusion in Lattices of the Zero-Power Plutonium Reactor Experiments. Gelbard, E.M. July, 1974 Nucl. Sci. Eng., 54(3), 341-352 A Theory for Pulsed-Neutron Experiments in Reactor A Theory Los ----Lattices, Van Binnebeek, J.J. July, 1974 Nucl. Sci. Eng., 54(3), 353-356 A Solution of the Milne Problem Using Full-Range Completeness of Case's Eigenfunctions. Loyalka, S.K. July, 1974 Nucl. Sci. Eng., 54(3), 357-360 Axial Neutron Streaming in Gas-Cooled Past Reactors. (Tech. Note) Kohler, P.; Liqou, J. July,1974 Nucl. Technology, 23(1), 87-93 Fast Neutron Flux Measurement Using Gas-Flow Techniques. Brown, D.P.; Spear, N.G. July, 1974 Phys. Med. Biol., 19(3), 329-340 Dose to the Urinary Bladder from Radionuclides in Urine. Unnikrishnan, K. May, 1974 Shimadzu Hyoron, 30 (2-3), 175-180 (In Japanese) Study on Shielding for Betatron. 2. Spectra of Primary and Transmitted X-Rays Through Various Shielding Materials for 4 to 30 My X-Rays. Maruyama, T.: Sakata, S.; Kumamoto, Y.; Fukuhisa, K.; Hashizume, T. September, 1973 Strahlentherapie, 147(2), 201-210 (In German) Guidelines for Operation of Accelerator Plants in Medicine. Stieve, F.E. 1974

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Activation. Lessler, R.M.; Alley, W.E.; Graen, J.B. Lawrence Livermore Laboratory, Livermore, California AVAIL: NTIS

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IL-TM-4273 July 1973 APSAI APSAI: A Computer Code for Plotting Fluxes and Absorption Densities Generated by the ANISN Code. Sahin, S. Oak Ridge National Laboratory, Oak Ridge, Tennessee ORNL-TM-4273 OBNL-TH-4601 June 1974 CRYSTAL BALL CRYSTAL BALL - A Computer Program for Determining Neutron Spectra from Activation Measurements. Kam, F.B.K.; Stallmann, F.W. Oak Ridge National Laboratory, Oak Ridge, Tennessee AWRE-0-30/73 1973 DIFFAL DIFFAL: A Computer Program for Prediction of Atmospheric Dispersion of Particulate Debris. Robson, A. UKAEA Weapons Group, Atomic Weapons Research Establishment, Aldermaston, England IR-P10-7366
 Express Program for Processing BPOS Spectra. Fitting
 Isolated and Weakly Overlapping Peaks.
 Eler, G.: Gopych, P.M.: Vinel, G.V.: Khabenikht, V.:
 Vylova, L.A.
 Joint Institute for Nuclear Research, Dubna, USSR
 AVAIL: NTIS JINR-P10-7366 L-51493 November 1973 ETRANMS ETRANMS - A One Dimensional Monte Carlo Electron/Photon Transport Code for Multimaterial Targets. UCBL-51493 An Sport code for an analysis of the second SLL-73-5281: CONF-730626-2 FEMESH: A Finite Element Code Preprocessor. Gabrielson, V.K. PENESH Sandia Laboratories, Livermore, California AVAIL: NTIS ABEW-M-843 December 1968 FRESCO; HURAL Calculation of Neutron Spectra and Group Averaged Cross-Sections Using the Computer Programmes FRESCO and MURAL. Macdougall, J.D.; Ross, R.W.; Rowlands, J.L. UKAEA Reactor Group, Atomic Energy Establishment, Winfrith, England AVAIL: NTIS AEEW-M-848 M-848 June 1968 FRESCO: MURAL Calculation of Resonance Shielding Factors in FRESCO and MUBAL. Macdougall, J.D.; Rowlands, J.L. UKAEA Reactor Group, Atomic Energy Establishment, Winfrith, England AVAIL: NTIS

EUR-5048 February 1974 ISOTEX ISOTEX: Computer Code for Calculation of Isotopes Concentrations and Ratios of Concentrations. Sola, A. Commission of the European Communities, Luxembourg and Ispra, Italy AVAIL: NTIS

- AD-746461; MR-7017; DNA-2890-F June 1972 BCTC III Automatic Computation of Importance Sampling Functions for Monte Carlo Transport Codes: Phase III. Final Report. Kalos, M.H.; Steinberg, H.; Troubetzkoy, E. Mathematical Applications Group, Inc., Elmsford, New York AVAIL: NTIS
- BT/FI-(72)31 August 1972 MENTE Convergence of the Integral Transform Method for the Solution of the Mono-Energetic Neutron Transport Equation. Boffi, V.C.; Molinari, V.G.; Premuda, F.; Trombetti, T. Comitato Nazionale Per L'Energia Nucleare, Rome, Italy AVAIL: NTIS
- SC-BR-710212 September 1971 NSIE Numerical Solutions of Integral Equations and Curve Fitting. Biggs, Frank: Amos, D.E. Sandia Laboratories, Albuguergue, New Mexico
- KFTI-71-26 Photonuclear Reaction Cross-Section Calculations. Kuz'menko, V.S.; Ranyuk, Yu.N. Akademiya Nauk Ukrainskoi SS8, Kharkov AVAIL: INIS
- LA-5486-MS February 1974 BEAPHY Standard Interface Files and Procedures for Reactor Physics Codes, Version III. Carmichael, B.M. (comp.) Los Alamos Scientífic Laboratory, New Mexico AVAIL: NTIS
- PTB-ND-4 June 1973 BOSMN Program for Calculating the Response Function of Organic Scintillators for Incident Monochromatic Neutrons in the Energy Range from 0.05 to 20.0 MeV. Dietze, G. Physikalisch-Technische Bundesanstalt, Brunswick, F. R. Germany AVAIL: NTIS
- JINE-D6-7094, p. 175 Program for Processing Spectra in FORTRAN. Avramov, S.R. Joint Institute for Nuclear Research, Dubna, USSR
- LA-DC-13275 November 1971 SLPT Simulation of Low-Energy Photon Transport and the Simulation of the Adjoint Neutron Transport Equation With Monte Carlo. Carter, L.L.; Cashwell, B.D.; Schrandt, R.G. Los Alamos Scientific Laboratory, New Mexico

AK-P-109 SNDIR Code for Neutron Flux Multigroup Calculations in One-Dimensional Geometry with Sn Method. Zadvornykh, Z.V.; Yaroslavtseva, L.N. Scientific Research Institute of Atomic Reactors, Dimitrovgrad, USSR ALPHA: ALGOL-60 AVAIL: INIS 1972 NIIAR-P-159 SNDIR I-73-57 June 1973 SPECTRANS-2 SPECTRANS-2: A Modified Computer Code for Standardizing Neutron Spectra. Palfalvi, J. Kozponti Fizikai Kutato Intezet, Budapest, Hungary KFKI-73-57 AVAIL: NTIS RI-M-4874 July 1972 ST-1; ST-2 Computer Codes ST-1 and ST-2 for Calculation of Neutron Energy Spectrum in a Fuel-Rod Cell. Toba, J.; Naito, Y. Japan Atomic Energy Research Institute, Tokyo, Japan JAERI-M-4874 AVAIL: NTIS September 1973 STRAINT AWRE-0-35/73 Calculation of Neutron Angular Plux Emerging From Shells Natural Uranium Surrounding a Central Source. West, P.H. UKAFA Weapons Group, Atomic Weapons Research Establishment, of Aldermaston, England AVAIL: NTIS 75 January 1972 STSD Space- and Time-Dependent Slowing Down in Heavy Media. Weisbin, C.B.; Forster, R.A.; Smith, D.B.; Corngold, N. Los Alamos Scientific Laboratory, New Mexico LA-4775 D-16393 November 1973 TART TART Calculations Using the New 2021 Energy Group Cross Section Library Compared with B. Czirr's Resonance Self-Shielding Experiments. UCID-16393 Bacon, L. Lawrence Livermore Laboratory, Livermore, California AVAIL: NTIS ORNL-TM-4451 January 1974 TECALC TECALC: A Program to Calculate Compton, Coherent, and Photoelectric Mass Attenuation Coefficients for Photons with Energies Less than 1 MeV and to Assist in the Evaluation and Formulation of Photon-Equivalent Materials. Stansbury, P.S. Oak Ridge National Laboratory, Oak Ridge, Tennessee BASIC PDP-10 AVAIL: NTIS -73-924 February 1974 TELIC TELIC: A Flexible, Two-Dimensional Simulation Code for Laser and Electron Beam Target Studies. Lane, F.; Widner, M. Sandia Laboratories, Albuquerque, New Mexico SLA-73-924 AVAIL: NTIS

- SLA-73-1026 March 1974 TIGER TIGER: A One-Dimensional, Multilayer Blectron/Photon Monte Carlo Transport Code. Halbleib, J.A.: Yandevender, W.H. Sandia Laboratories, Albuquerque, New Mexico
 C00-2060-16 February 1972 THMS1 Applications of the Transmission Matrix to Radiation Shielding. Final Report. Rohach, A.H. Lowa State University, Ames, Iowa
- STAN-CS-73-334 January 1973 URAND URAND: A Universal Bandom Number Generator. Malcolw. M.A.; Moler, C.B. Stanford University, California FORTRAN

. .

AEEW-R-817 Eay 1973 XENFER Calculation of Nuclear Reactor Spatial Transfer Functions Using the Computer Programme XENFER. Owens, D.H. Atomic Energy Establishment, Winfrith, England AVAIL: NTIS