

# RSIC Newsletter



RADIATION SHIELDING INFORMATION CENTER

**OAK RIDGE NATIONAL LABORATORY**

OPERATED BY UNION CARBIDE CORPORATION • FOR THE U.S. ATOMIC ENERGY COMMISSION

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*Thinking isn't to agree or disagree - that's voting.*

*...Robert Frost*

## COMPUTING FACILITY INFORMATION NEEDED

Appended to this issue of the Newsletter is a form for your use in furnishing to RSIC information concerning the computing hardware and software being used at your installation. With this knowledge, the RSIC staff can better service the requests from your installation and can more suitably advise on operability of any given code package at your installation. PLEASE RETURN the completed form to the Radiation Shielding Information Center, Oak Ridge National Laboratory, P. O. Box X, Oak Ridge, Tennessee 37830.

## AMERICAN NUCLEAR SOCIETY TO MEET IN SAN FRANCISCO

The ANS winter meeting will be in San Francisco, November 11-15, 1973. Sessions sponsored by the Shielding and Dosimetry Division or of special interest to shielding people include: *Development of Data for Dosimetry Measurements* (Monday AM, all papers invited), *Experimental Radiation Shielding* (Monday PM), *Development of Standardized Spectra, Cross Section Data, and Computational Techniques for Materials Dosimetry* (Monday PM), *Environmental Radiation Dosimetry I and II* (Tuesday), *Finite Element Methods for Solution of Neutron Diffusion and Transport Problems* (Tuesday PM), *Shielding Cross Sections* (Wednesday AM), *Neutronics of Fusion Reactors* (Wednesday AM), *Reactor Radiation Shielding* (Wednesday PM), *Cross Section Processing and Applications* (Wednesday PM), *Weapons Radiation Shielding* (Thursday AM), *Monte Carlo and Transport Methods* (Thursday PM) *Photon and Charged Particle Transport* (Thursday PM).

The Shielding Standards Subcommittee, ANS-6, will meet Monday Afternoon at 4:00 PM in the Terrace Room of the Hilton. New business includes forming two new working groups on standards for design stage estimates of radiation from nuclear power plants and radiation level zoning in nuclear power plants. All those interested are invited to attend.

## MACHINE INDEPENDENT FORTRAN RANDOM NUMBER GENERATOR AVAILABLE

David Irving of Science Applications, Inc., La Jolla, California, has developed a FORTRAN random number generator which is machine independent. Heretofore, FORTRAN or Assembly Language random number generators for Monte Carlo codes have been machine dependent and lead to some nuisance in translating codes from one machine to another. The subroutine is available from RSIC upon request.

## CHANGES TO THE DATA LIBRARY COLLECTION

Two data packages have been changed during the month of September:

- DLC-5/HALLMARK Discrete Ordinates and Monte Carlo Results of Neutron and Secondary Gamma-Ray Transport in Air-Over-Ground Geometry data package has been extended by Science Applications, Inc., Huntsville, Ala., and Oak Ridge National Laboratory. We have added 22 neutron, 18 gamma-ray group cross sections for air and ground which were used in CCC-89/DOT to generate part of the DLC-5/HALLMARK data. These can be used as a reference set of data for those who wish to perform similar calculations or perhaps augment the results which were published by E. A. Straker in ORNL-4289, Vol. II. The data are available on cards (614) upon request.
- DLC-10/AVKER Neutron Kerma Response Function Data Library Package has been updated to correct an error in Subroutine AVKER. The dimensions of the variables DUME and DUMKER must be increased from 120 to be  $\geq 500$  (perhaps as high as 816). This error was discovered by Gerald P. Lahti, Sargent and Lundy, and was verified by M. Solomito. The RSIC package reflects this change as of 9-6-73.

## COMPUTER CODE COLLECTION CHANGES

Several changes were made to the code collection during the month, including additions, modifications, and corrections.

- CCC-107/ETRAN The Monte Carlo Code System for Electron and Photon Transport Through Extended Media Code Package has been extended to include 18G, a code version designed to handle cylindrical targets. The package now includes 16D (one dimensional ETRAN), 18G (cylindrical), and auxiliary codes INCLUDE and DATAPAC 6 (cross section handling code), a data library, DATATAPE 2, and input and output from running sample problems. FORTRAN IV, IBM 360. Reference: NBS-9836, -9837, and Informal Notes. Contributor: Center for Radiation Research, National Bureau of Standards, Washington, D.C.

- CCC-120/SPACETRAN The package, Dose Calculations at Detectors at Various Distances from the Surface of a Cylinder, has been extended by the addition of SPACETRAN III: Integration of DOT III Surface Angular Fluxes. In addition, the package contains SPACETRAN I: Integration of ANISN Leakage Data and SPACETRAN II: Assumed Powers of Cosine Angular Distribution. FORTRAN IV, IBM 360. References: ORNL-TM-2592 and CTC-INF-952. Contributor: Oak Ridge National Laboratory.
- CCC-129/TWOTRAN-SPHERE Multigroup Two Dimensional Discrete Ordinates Transport Code in Spherical Geometry, a modification of the general-geometry TWOTRAN program, contributed by the Los Alamos Scientific Laboratory, Los Alamos, New Mexico. FORTRAN IV, CDC 6600/7600. References: LA-4567 and -4432.
- CCC-144/TIMOC The code package, Monte Carlo Three-Dimensional Neutron Transport Code System, has been updated to reflect more recent code development named TIMOC-72. The cross section handling code, CODAC, handles early ENDF formats and a replacement has been requested but not yet received. FORTRAN IV, IBM 360/370. References: EUR-4519.e and EUR-4521.e. Contributor: CCR Euratom, Ispra (Varese), Italy, through the OECD Nuclear Energy Agency's Computer Programme Library.
- CCC-203/MORSE-CG The IBM 360 package, MORSE-CG (CCC-203C), has been extended by the addition of DOMINO, a general purpose code for coupling discrete ordinates and Monte Carlo calculations. DOMINO is also packaged as a stand-alone code package (PSR 64). CCC-203 contains the general purpose Monte Carlo neutron and gamma-ray transport code MORSE with MAGI's combinatorial geometry. Input and output from sample problems run through the entire system are also packaged. DOMINO is described in ORNL-4853. Contributor: Oak Ridge National Laboratory.
- CCC-218/EZTRAN Monte Carlo Code System for Electron and Photon Transport Through Slabs, contributed by Sandia Laboratories, Albuquerque, New Mexico. References: SC-DR-71 0598, NBS Reports 9836 and 9837. EZTRAN represents Sandia modifications and extensions made on the base of the NBS ETRAN 15 Monte Carlo Code and DATAPAC 4 to perform deep penetration electron transport studies. FORTRAN IV, CDC 6600.

- CCC-219/PIPE Numerical Gamma-Ray Transport Code for Plane/Spherical Geometry based on the BIGGI series of code development has been contributed by CCR Euratom, Ispra (Varese) Italy, through the OECD NEA Computer Programme Library. FORTRAN IV, IBM 360/370. Reference: ISPRA-1437.
- CCC-220/LUIN Analytical Straight-Ahead Transport Code - Calculation of Gamma-Ray Spectra, Fluxes and Ionization in the Earth's Atmosphere - contributed by the USAEC Health and Safety Laboratory, New York, New York. FORTRAN IV, CDC 6600. Reference: HASL-275.
- CCC-221/SLDN A Code System for Shielding Calculations by the Method of Invariant Imbedding operable on the GE-series (635 and 685) computers has been contributed by Tokyo Shibaura Electric Co., Ltd. and the Power Reactor and Nuclear Fuel Development Corporation through the Japan Atomic Energy Research Institute, Ibaraki, Japan. The GE series of programs are available as CCC-221A. Conversion to the IBM 360 computers is currently underway in RSIC. The codes in the system can be operated independently. The reflection and penetration of neutrons and gamma rays, including gamma rays produced by  $(n,\gamma)$  reactions, through multilayer slabs can be calculated based on the multigroup cross sections generated from ENDF/B with an appropriate combination of the programs. They are: SLDN-LTA-2: Neutron Multigroup Constant Generator; SLDN-LTB-2: Generator of Elastic Scattering Kernel of Neutrons and Gamma Rays; SLDN-SL-2: Calculation of the Reflection and Transmission Functions of Radiations for Homogeneous Slabs; SLDN-ML-2: Calculation of Penetration of Radiation for Multilayer Slabs; SLDN-DF-2: Neutron Multigroup One Dimensional Diffusion Code for Shielding Calculations. References: J201 71-08, J201 71-09, J201 71-10, J201 71-11, J201 71-12, and J201-72-06, all written in English.
- CCC-222/TWOTRAN II Two Dimensional Multigroup Discrete Ordinates Transport Code in  $(x,y)$ ,  $(r,\theta)$  and  $(r,\phi)$  Geometries, contributed by Los Alamos Scientific Laboratory. Both regular and adjoint, inhomogeneous and homogeneous problems subject to vacuum, reflective, periodic, white or input specified boundary flux conditions are solved. General anisotropic scattering is allowed and anisotropic inhomogeneous sources are permitted. FORTRAN IV, CDC 7600 and IBM 360 versions. Other code development in the TWOTRAN series available: CCC-129/TWOTRAN-SPHERE and CCC-195/TWOTRAN-(PN, FC, VW) - for spherical harmonics, first collision source, and variable weight respectively. References: LA-4848 MS, LA-4774, LA-4600. LA-4567, LA-4432, and LA-4058.

- PSR-15/UKE III Cross Section Format Translator - UKNDL to ENDF/B - revised to translate UK data library into Version III of ENDF format, contributed by UCND Computer Sciences Division and Oak Ridge National Laboratory. FORTRAN IV; IBM 360. Reference: ORNL-TM-2880 (Rev.)
- PSR-63A/AMPX Modular Code System for Generating Coupled Multigroup Neutron Gamma-Ray Cross Section Libraries from ENDF/B, contributed by UCND Computer Sciences Division and Oak Ridge National Laboratory. The AMPX system combines DRIVER (supervisor module), XLACS (neutron processor), SMUG (gamma-ray interaction processor), CHOX (interface module), NITAWL (resonance self-shielding and data generation), LAPHNGAS (gamma-ray production processor), XSDRNPM (1-D  $S_n$  transport for spatial weighting), and subroutines needed for anisotropic matrices. Two sample problems are packaged. FORTRAN IV; IBM 360 and CDC 6600. Reference: ORNL-TM-3706 (AMPX-1).
- PSR-64/DOMINO General Purpose Code for Coupling Discrete Ordinates and Monte Carlo Radiation Transport Calculations, contributed by the Oak Ridge National Laboratory. DOMINO is considered to be a useful stand-alone discrete ordinates - Monte Carlo coupling code and, therefore, is also available separate from MORSE. FORTRAN IV; IBM 360. Reference: ORNL-4853.
- PSR-65/APSAI Activity Calculations and Plotting of Neutron or Gamma-Ray Spectra Generated by Discrete Ordinates Code ANISN (CCC-82), contributed by Oak Ridge National Laboratory and Karadeniz Technical University, Trabzon, Turkey. FORTRAN IV, IBM 360. Reference: ORNL-TM-4273.

#### PERSONAL ITEMS

The following changes of address have been received by RSIC: F. Eric Haskin from the Department of Nuclear Engineering, University of Arizona, to Bechtel Associates Professional Corp., Ann Arbor, Mich.; John H. Weiler from the Essex Service Co., Houston, Texas, to Offshore Power Systems, Jacksonville, Fla.; Steven J. Nathan from Ralph M. Parsons Company, Los Angeles, to NUS Corporation, Rockville, Md.

#### VISITORS TO RSIC

Visitors to RSIC during the month of September were: L. C. Byrd, K-25 Operations Division, UCND; T. J. DeRosa, Picatinny Arsenal, Dover, N. J.; Young Ku Yoon, Korea Atomic Energy Research Institute, Seoul, Korea; F. J. Patti, Burns & Roe, Inc., Hempstead, N.Y.

SEPTEMBER ACCESSION OF LITERATURE

REACTOR AND WEAPONS SHIELDING

- ACI-Special Pub.34, pp.1195-1224; CONF-701007-(Vol.3),  
pp.1195-1224  
Iron-Serpentine Concrete.  
Davis, H.S.  
October 5, 1970  
Detroit, American Concrete Institute (1972)
- ACI-Special Pub.34, pp.1225-1241; CONF-701007-(Vol.3),  
pp.1225-1241  
Serpentine Concrete for Czechoslovakian A-2 Reactor  
Shielding.  
Honig, A.; Vesely, F.; Samalikova, M.; Stoces, B.;  
Havranek, M.  
October 5, 1970  
Detroit, American Concrete Institute (1972)
- ACI-Special Pub.34, pp.1243-1253; CONF-701007-(Vol.3),  
pp.1243-1253  
On Properties of Magnetite and Serpentine Concrete at  
Elevated Temperatures for Nuclear Reactor Shielding.  
Ohgishi, S.; Miyasaka, S.; Chida, J.  
October 5, 1970  
Detroit, American Concrete Institute (1972)
- AERE-R-7374  
A UKAEA Evaluation of Displacement Damage Models for Iron.  
Etherington, E.W.; Bramman, J.I.  
March, 1973  
NTIS
- ANL-7925  
Applications of Finite Element Methods in Reactor  
Mathematics Numerical Solution of the Neutron Diffusion  
Equation.  
Kaper, H.G.; Leaf, G.K.; Lindeman, A.J.  
February, 1972  
NTIS
- ANL-7928  
Results of Monte Carlo Calculations for ZPPR Assembly 2.  
Fillmore, P.L.  
March, 1972  
NTIS
- ANU-P-576  
Radiation and Fallout.  
Titterton, E.W.  
No Date  
Dep., NTIS (U.S. Sales Only) \$3.00
- BNL-50374; ENDF-185  
Evaluated Neutron Cross Sections for the Stable Isotopes  
Of Xenon.  
Bhat, M.R.; Mughabghab, S.F.  
February, 1973  
NTIS

- BNL-50379; ENDF-186  
Neutron and Gamma Ray Production Cross Sections for  
Silicon.  
Bhat, M.R.; Goldberg, M.D.; Kinsey, R.R.; Prince, A.;  
Takahashi, H.  
March, 1973  
NTIS
- BNL-50383; ENDF-163  
Evaluated Neutron Cross Sections for 107-Ag, 109-Ag,  
and 133-Cs.  
Bhat, M.R.; Prince, A.  
April, 1973  
NTIS
- BNL-50387; ENDF-187  
Multi-Level Effects in Reactor Calculations and the  
Probability Table Method.  
Bhat, M.R. (Ed.)  
April, 1973  
NTIS
- BNL-50388; ENDF-190  
An Analysis of the 239-Pu Neutron Cross Sections from  
20 keV to 20 MeV.  
Prince, A.; Drake, M.K.; Hlavac, P.  
April, 1973  
NTIS
- BNWL-SA-4591  
Application of Pulsed Neutron Source Techniques to  
Plant Systems.  
Bleman, S.R.; Clayton, E.D.  
June, 1973  
NTIS
- CEA-CONF-2213 (In French); CONF-721018-10 (In French)  
Computational Methods for Graphite Gas Cooled Reactors  
Shielding: Comparison with Shielding Measurements Made  
During Start-Up Tests.  
Brisbois, J.; Duco, J.; Bourdeau, F.; Chapus, J.  
No Date  
Dep., NTIS (U.S. Sales Only) \$3.75
- CEA-N-1587 (In French)  
Spectral Distributions of Cobalt Therapy Units.  
Legrand, J.; Morel, J.  
February, 1973  
Dep., NTIS (U.S. Sales Only) \$3.25
- CONF-721018-18  
Discrete Ordinates-Numerical Integration Method for  
Neutron Transport Equation in One and Two Dimensional  
Geometries.  
Takeuchi, K.; Yamaji, A.  
1972  
Dep., NTIS (U.S. Sales Only) \$9.00

- CONF-700939- (Vol.2)  
Radiation Dosimetry. Vol.2. Proceedings of the  
International Summer School on Radiation Protection,  
Caviat, Yugoslavia, September 21-30,1970.  
Miric, I. (Ed.)  
1971  
Belgrade; Boris Kidric Institute of Nuclear Sciences  
(1971)
- CONF-721018-23  
Shielding Optimization by Differential Dynamic  
Programming Method and Experiment.  
Kanai, Y.; Yamaji, A.; Takeuchi, K.  
1972  
Dep., NTIS (U.S. Sales Only) \$3.00
- CONF-721111-3  
Review of ORNL Fusion Reactor Technology Studies  
(November, 1971-November, 1972).  
Steiner, D.  
1972  
Dep., NTIS
- CONF-730301, pp.1095-1104  
Photonuclear Reactions in Reactors.  
Francis, N.C.; Lubert, M.; Mendelson, M.R.; Block, R.C.  
1973  
Ernest O. Lawrence Livermore Lab., Univ. of Calif.,  
Livermore, Calif.
- CONF-730813-1  
Helium-Ion Irradiation of Vanadium, Niobium.  
Das, S.K.; Kaminsky, M.  
1972  
Dep., NTIS
- CONF-730814-1  
Physical Dosimetry Involved in Making Dose Estimates  
for Population Exposures.  
Snyder, W.S.  
1973  
Dep., NTIS \$3.50
- CONF-730907-3  
Absorbed Dose to Selected Internal Organs from Typical  
Diagnostic Exposures.  
Poston, J.W.; Warner, G.G.  
1973  
Dep., NTIS \$3.00
- COO-2262-1; MITNE-145  
Finite Element Method Applied to Neutron Diffusion  
Problems.  
Deppe, L.O.; Hansen, K.F.  
February, 1973  
Dep., NTIS \$8.25



- COO-3028-5; CONF-730602-2  
Radiation Damage Effects on Hydrogen Permeation in  
Fusion Reactors.  
Perkins, H.K.; Axtmann, R.C.  
1972  
Dep., NTIS \$3.00
- COO-3496-27  
Threshold Photofission: Theory and Experiment.  
Huizenga, J.R.; Britt, H.C.  
March, 1973  
NTIS
- DNA-3129F  
Preliminary Evaluation of the Neutron and Photon  
Production Cross Sections for Copper.  
Drake, M.K.; Fricke, M.P.  
May 11, 1973  
Intelcom Rad Tech, San Diego, Calif. 92138
- DNA-3130F  
Preliminary Evaluation of the Neutron and Photon  
Production Cross Sections for Magnesium.  
Drake, M.K.; Fricke, M.P.  
May 11, 1973  
Intelcom Rad Tech, San Diego, Calif. 92138
- DP-MS-72-72; CONF-721209-3  
Advances in Personnel Neutron Dosimetry.  
Hoy, J.E.  
No Date  
Dep., NTIS \$3.00
- DP-MS-73-19; CONF-730414-6  
Current Trends in the Development of Computer Systems  
for Reactor Analysis.  
Honeck, H.C.  
No Date  
Dep., NTIS \$3.00
- EIR-231  
Comparison of the Space Legendre Polynomials Approach  
with Other Methods Used for the Solution of the Integral  
Transport Equation.  
Stepanek, J.  
December, 1972  
Dep., NTIS (U.S. Sales Only) \$4.25
- GULF-GA-B-12071  
Review of Fission Product Yields and Cross Sections.  
Mathews, D.R.; McGehee, B.G.  
September, 1971  
Dep., NTIS
- HEDL-SA-548; CONF-730910-2  
EBR-II Dosimetry Tests.  
Jackson, J.L.; Kellogg, L.S.; McElroy, W.N.; Ulseth, J.A.  
1972  
Dep., NTIS \$3.00

- HEDL-SA-556; CONF-730910-3  
Fast Reactor Flux-Spectral Characterization.  
McElroy, W.N.  
1973  
Dep., NTIS \$3.25
- HEDL-TME-73-30; ENDF-171  
Neutron Resonance Spacings for Spherical Nuclei.  
Schmittroth, F.  
January, 1973  
Hanford Eng. Dev. Lab., P.O. Box 1970, Richland, Wa. 99352
- ITJ-30  
Absorption of the Gamma Radiation in Inhomogeneous Media.  
Gyurcsak, J.; Uniastowski  
November, 1972  
Dep., NTIS (U.S. Sales Only) \$5.50
- JUL-931-RG (In German)  
Neutronic Studies of D-T Fusion Reactor Blankets.  
Brockmann, H.  
March, 1973  
NTIS
- KEES-SR-113  
Validity of Scale-Modeling for Gamma-Ray Attenuation.  
Verser, F.A.; Donnert, H.J.  
September 1, 1973  
Dept. of Nucl. Eng., Kansas State University,  
Manhattan, Kansas 66506
- LBL-1324  
Radiative Transfer in a Dense Water Vapor Atmosphere.  
Coakley, J.A., Jr.  
September, 1972  
Dep., NTIS
- LA-5172  
Evaluated Neutron-Induced Cross Sections for <sup>239</sup>Pu and  
<sup>240</sup>Pu.  
Hunter, R.E.; Stewart, L.; Hiron, T.J.  
June, 1973  
NTIS
- LA-5253-MS; USNDC-6  
Compilation of Requests for Nuclear Data.  
Stewart, L.; Moore, M.S.; Motz, H.T. (Eds.)  
June, 1973  
NTIS
- LA-TR-73-32; CONF-730302-14  
Problems of Measuring Nuclear Constants for Thermonuclear  
Reactors.  
Kuz'min, E.A.; Ogloblin, A. A.; Sidorov, N.I.;  
Paiziev, A.R.  
1972  
Dep., NTIS \$4.00

- LA-UR-73-918; CONF-730713-1  
Comparative Efficiency of Certain Finite Element and  
Finite Difference Methods for a Hyperbolic Problem.  
Swartz, B.; Wendroff, B.  
1973  
Dep., NTIS \$3.00
- MLM-1982  
A Bifold Nuclear Power Source.  
Wolfe, R.A.; Stubbins, W.F.  
August 15, 1973  
Mound Laboratory
- NASA-TN-D-7292  
An Iterative Solution of an Integral Equation for  
Radiative Transfer by Using Variational Technique.  
Yoshikawa, K.K.  
August, 1973
- NASA-TN-D-7384  
Net Radiation Method for Enclosure Systems Involving  
Partially Transparent Walls.  
Siegel, R.  
August, 1973
- ORNL-4897  
Cross Sections for Atomic Displacements in Solids by  
Fast Electrons.  
Oen, O.S.  
August, 1973  
NTIS
- ORNL-TM-3911  
Description and Safety Analysis of Physical Examination  
Hot Cells Building 3025.  
King, E.M.; Walls, A.A.  
November, 1972  
Dep., NTIS
- ORNL-TM-3958; Thesis  
Incorporation of Outer Iteration Techniques into the  
Task Algorithm.  
Lillie, R.A.  
November, 1972  
NTIS
- ORNL-TM-4014  
One-Dimensional Calculations in Support of the Reactivity  
Surveillance Procedures for the Fast Test Reactor.  
Planagan, G.F.; Simpson, D.B.; Buhl, A.R.  
April, 1973  
AT-(TIC)
- ORNL-TM-4060  
Shielding Against the Neutron Produced When 400-MeV  
Electrons are Incident on a Thick Copper Target.  
Alsmiller, R.G., Jr.; Barish, J.  
April, 1973

- ORNL-TM-4257; ENDF-191  
ADLER-III: A Program to Calculate Cross Sections from  
Adler-Adler Resonance Parameters.  
Wright, R.Q.  
June, 1973  
RSIC - Packaged as PSR-58
- RISO-119  
Nuclear Heat Dose Rate Separation by Calculation and  
Measurement.  
Haack, K.  
March, 1972  
Danish Atomic Energy Commission Research Establishment,  
Riso
- RT/PI-(72)44 (In Italian)  
Bringing Up to Date, Using, and Controlling the  
Multigroup Cross Section Library for Fast Reactor  
Calculations.  
Cosimi, M.; Salvatores, M.  
September 26, 1972  
Dep., NTIS (U.S. Sales Only)
- SLA-73-562  
Approximation for X-Ray Cross Sections.  
Sullivan, T.D.  
July, 1973  
Dep., NTIS \$3.50
- SLA-73-5352  
Laser Fusion.  
Palmer, R.E.  
No Date  
Dep., NTIS \$3.00
- TUBIK-16 (In German)  
Theory and Calculation Programs for Solution of the  
Neutron Transport Equation According to PN and the Double  
PN Method for Slab Geometry.  
Schumann, U.; Bartsch, G.  
1972  
Dep., NTIS (U.S. Sales Only) \$10.25
- UCRL-74792; CONF-730813-2  
Proton Simulation of a Fusion Reactor Neutron  
Environment.  
Logan, C.M.; Anderson, J.D.  
June, 1973  
Dep., NTIS \$3.00
- UCRL-74855; CONF-730647-1  
Neutron Dosimetry Studies at LLL.  
Griffith, R.V.  
July 9, 1973  
Dep., NTIS \$3.00

- UCID-16313  
Steps Toward Importance Sampling for MCMEG.  
Loewe, W.E.  
July 6, 1973  
Dep., NTIS \$3.00
- UCRL-51,412  
Field Spectrometric Measurements of Radionuclide  
Concentrations and External Gamma Exposure Rates at  
the Nevada Test Site. A Demonstration Study.  
Anspaugh, L.R.; Phelps, P.L.; et al.  
July 5, 1973  
NTIS
- WAPD-TM-1076  
On the Use of Residuals for the Estimation of Error in  
Approximate Synthesis Solutions. (LWBR Development  
Program).  
Yasinsky, J.B.  
June, 1973  
NTIS
- Archive for National Mechanics and Analysis, 50(4), 290-292  
Solution of a Nonlinear Boltzmann-Equation for  
Neutron-Transport in L1 Space.  
Pao, C.V.  
1973
- Atomic Energy Rev., 10(4), 529-636  
Simultaneous Evaluation of the 239-Pu Fission  
Cross-Section Ratio in the Fast Neutron Energy Region.  
Byer, T.A.  
December, 1972
- Health Phys., 25(2), 163-167  
A Storage and Handling Facility for Californium-252  
Medical Sources.  
Veerling, J.P., Jr.; Oliver, G.D., Jr.; Moore, E.B.  
August, 1973
- Health Phys. 25(2), 175-176  
Neutron Cross Sections for the Egyptian Marble.  
Gomaa, M.A.; El-Shinawy, R.M.K.; Mohamed, H.O.  
August, 1973
- Health Phys., 25(2), 176-179  
On the Discussion of the Quality Factor.  
Keyrim-Markus, I.B.; Popov, V.I.  
August, 1973
- Health Phys., 25(2), 191-194  
Depth Dose Distributions Due to 14 MeV Neutrons.  
McGinley, P.H.  
August, 1973

- Health Phys., 25(3), 334  
Using Measured Scattered and Leakage X-Ray and  
Gamma-Ray Spectra for Shielding Design. (Meeting Abstract)  
Bell, J.H.; Haggener, R.G.; Payne, W.H.; Zanca, P.  
September, 1973
- J. Nucl. Energy, 27(2), 121-122  
Boundedness of Integral Transport Operators with  
Fission.  
Ronen, Y.  
February, 1973
- J. Nucl. Energy, 26(7), 367-378  
Integral Transform Method for 2-Dimensional and  
3-Dimensional Neutron-Transport Problems.  
Sahni, D.C.  
1972
- J. Nucl. Energy, 27(4), 247-262  
Calculation of Neutron Slowing Down Spectrum in Water  
Using Green's Function.  
Bhatti, R.A.  
April, 1973
- J. Nucl. Sci. Technol. (Tokyo), 10(5), 284-291  
Solutions of Monoenergetic Time-Dependent  
Neutron-Transport Equation in Slab Geometry.  
Yanagishi, T.  
1973
- Kernenergie, 16(7), 218-221  
New Spectral Representation in Neutron-Transport  
Theory.  
Collatz, S.  
1973
- Nuclear Data Tables, 11(1), 1-126  
Reaction List for Charged-Particle-Induced Nuclear  
Reactions.  
McGowan, F.R.; Milner, W.T.  
1972
- Nuclear Data Tables, 11(No.8-9), 621-826  
Neutron Activation Cross Sections - Measured and  
Semiempirical.  
Alley, W.E.; Lessler, R.M.  
July, 1973
- Nucl. Instrum. Methods, 101(3), 413-422  
Monte-Carlo Calculations for Thin NaI(Tl) Crystals at  
Energies Below 100 keV.  
Sharma, R.C.; Garg, S.P.; Somasundaram, S.  
1972
- Nucl. Instrum. Methods, 111(1), 157-168  
Accurate Formulas for Calculation of High-Energy  
Electron Bremsstrahlung Spectra.  
Matthews, J.L.; Owens, R.O.  
1973

- Nucl. Sci. Eng., 52(1), 12-22  
The Application of Phase-Space Finite Elements to the  
Two-Dimensional Neutron Transport Equation in X-Y Geometry.  
Miller, W.F., Jr.; Lewis, E.E.; Rossow, E.C.  
September, 1973
- Nucl. Sci. Eng., 52(1), 23-34  
Two-Dimensional Neutron Transport Calculations Using the  
Transport Matrix Method.  
Filippone, W.L.  
September, 1973
- Nucl. Sci. Eng., 52(1), 35-45  
Neutron Spectra from Deuteron Bombardment of D, Li,  
Be, and C.  
Weaver, K.A.; Anderson, J.D.; Barschall, H.H.; Davis, J.C.  
September, 1973
- Nucl. Sci. Eng., 52(1), 117-123  
Calculation of Gamma-Ray Buildup Factors Including  
the Contribution of Bremsstrahlung.  
Kuspa, J.P.; Tsoulfanidis, N.  
September, 1973
- Nucl. Sci. Eng., 52(1), 130-141  
Monte Carlo Simulation of Radiation-Induced Plasmas.  
Wang, B.S.; Miley, G.H.  
September, 1973
- Nucl. Technology, 18(1), 5-14  
Calculated Radiation Properties of Spent Plutonium  
Fuels.  
Bell, M.J.  
April, 1973
- Nucl. Technology, 18(1), 15-24  
The Analytically Determined Response of Silicon  
Detectors to a Polyenergetic Neutron Beam.  
Taherzadeh, M.  
April, 1973
- Nucl. Technology, 19(3), 202-203  
Neutron Yield from a Small High Purity 238-PuO<sub>2</sub> Source.  
Baur, J.K.; Butler, H.M.  
September, 1973
- Nuovo Cimento A, 13(1), 267-270  
Shell-Wise Photoelectric Cross-Sections of Gamma-Rays.  
Rao, K.S.; Rao, B.V.; Parthasaradhi, K.  
January 1, 1973
- Phys. Med. Biol., 17(6), 884  
Shielding to Fast-Neutrons Provided by Iron Barriers.  
(Abstract)  
Szabo, F.P.; Clifford, C.E.; Francillon, G.  
1972

Z. Phys., 262(5), 393-312  
Photoproduction of High-Energy Neutrons in Thick  
Targets by Electrons in Energy Range 150 to 270 MeV.  
Voneyss, H.J.; Luhrs, G.  
1973

Thesis  
The Transmission of Neutrons through Straight  
Cylindrical Ducts in Reactor Shields.  
Gibbs, D.C.C.  
Available in 1973 - Mf RSIC  
Royal Naval College; Greenwich, London

BOOK (In Russian)  
LECTURES ON NEUTRON TRANSPORT THEORY.  
Smelov, V.V.  
1972  
Moscow; Atomizdat (1972)



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