

RSIC Newsletter



RADIATION SHIELDING INFORMATION CENTER

OAK RIDGE NATIONAL LABORATORY

OPERATED BY UNION CARBIDE CORPORATION • FOR THE U.S. ATOMIC ENERGY COMMISSION

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*Men are disturbed not by things,
but by the view which they take of things.
...Epictetus (c.A.D. 50-120)*

LICENSING AND REGULATION COURSE TO BE OFFERED IN THE AUTUMN

The ALI-ABA course of study *Atomic Energy Licensing and Regulation-IV*, will be offered in Washington, D.C. (Washington Hilton), September 20-22, 1973. The registration fee is \$225.00.

For further information write or call Paul A. Wolkin, Director, or Donald M. Maclay, Assistant Director, ALI-ABA Joint Committee on Continuing Legal Education, 4025 Chestnut Street, Philadelphia, Pennsylvania 19104, or telephone (215) 387-3000.

COMPILATION FROM CERN AVAILABLE

Compilation of Cross Sections - I, π^- and π^+ Induced Reactions (CERN/HERA 72-1) by E. Bracci, J. P. Droulez, E. Flaminio, J. D. Hansen, and D. R. O. Morrison is a compilation of cross sections of reactions produced by negative and positive pions on targets of protons, neutrons, and deuterons. This is an updated version of CERN-HERA 70-5 and 70-7 and contains 40 per cent more data values than the earlier publications. Graphs of the variation of cross section with incident laboratory momentum are plotted. Values of the rate of decrease of cross section with incident momentum are given. The compilation is available in the Western Hemisphere and Far East from Lawrence Berkeley Laboratory, Berkeley, Calif. 94720; users elsewhere should order from CERN, Geneva, Switzerland.

CHANGES TO THE COMPUTER CODE COLLECTION

The following codes have been added to the collection. Requests should be accompanied by a full reel (2400') of magnetic tape and a statement as to how the tape should be written (density, whether blocked or unblocked, number of tracks).

CCC-210/NUGAM 2&3 Monte Carlo Prediction of Photon Transport Distributions, contributed by NUS Corporation, Rockville, Maryland, and NASA Goddard Space Flight Center, Greenbelt, Md. FORTRAN IV, IBM-360. Reference: NUS-786 Vols. 1, 2, 3.

CCC-211/EMERALD

Calculation of Activity Releases and Potential Doses from a Pressurized Water Reactor Plant, contributed by Pacific Gas and Electric Company, San Francisco, California. FORTRAN IV, IBM-360.

PERSONAL ITEMS

Gerald P. Lahti, formerly of NASA-Lewis Research Center, has recently accepted a position with Sargent and Lundy, Engineers, in Chicago. He has joined the Nuclear Safeguards and Licensing Division, which is responsible for the shielding, radiological safety, and licensing of nuclear power plants.

Ralph Best, formerly with Nuclear Fuel Services, is now with Allied Gulf Nuclear Service, Barnwell, S. C.

Oadis Jenkins, formerly manager of Liquid Metals Information Center (LMIC), is starting an information system at Burns and Roe, Oradell, N. J., as part of their LMFBF project.

Louise Hicks is the Oak Ridge Associated Universities (ORAU) summer student this year at RSIC. She comes to us from Talladega College, Talladega, Alabama.

Mary Nicholson, RSIC information specialist for the past two years, has accepted a library research position with the Tennessee Valley Authority (TVA) in Knoxville, Tennessee.

VISITORS TO RSIC

Visitors to RSIC during the month of June were: J. C. Courtney, Louisiana State University, Baton Rouge; H. Goldstein, Columbia University, New York, N. Y.; J. Pop-Jordanov, University of Belgrade, Yugoslavia; J. R. A. Lakey, Royal Naval College, London, England; S. Pearlstein, Brookhaven National Laboratory, Upton, N. Y.; J. G. Royen, OECD European Nuclear Energy Agency, Paris; R. E. Rexroad, U.S. Army Ballistic Research Labs, Aberdeen Proving Ground, Md.; L. Stewart, Los Alamos Scientific Laboratory, Los Alamos, N. M.; B. Wehring, University of Illinois, Urbana.

JUNE ACCESSION OF LITERATURE

The following literature cited has been ordered for review, and that selected as suitable will be placed in the RSIC Information Storage and Retrieval Information System (SARIS). This early announcement is made as a service to the shielding community. Copies of the literature are not distributed by RSIC. They may generally be obtained from the author or from a documentation center such as the National Technical Information Service, Dept. of Commerce, Springfield, Virginia 22151.

REACTOR AND WEAPONS SHIELDING

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R2 Measurements and Analysis of Gamma Heating in the
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Carlsson, R.; Larsson, L.G.
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- AECL-4080
Guide to Making Statistical Tests with Small Samples.
Jarvis, R.G.
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Yields in Thermal Neutron Fission. Some Results and
Recommendations Based on a Recent Evaluation.
Walker, W.H.
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- ANL-7807
Survey of Thermonuclear-Reactor Parameters.
Persiani, P.J.; Lipinski, W.C.; Hatch, A.J.
October, 1972
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- ANL-8001
A Review of the Chemical, Physical, and Thermal
Properties of Lithium that are Related to Its Use in
Fusion Reactors.
Maroni, V.A.; Cairns, E.J.; Cafasso, F.A.
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- AREAEE-153
Numerical Formulation of Multigroup and Discrete Sn
Methods for Reactor Analysis.
Nasser, S.F.
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Interaction Between the National Neutron Cross Section
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- BNWL-SA-4449 (Dr)
Statistical Study of Radiation Dose for Fishermen.
Beetle, T.M.
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- BRL-CR-100
Coupling of Forward and Adjoint Calculations to Solve
Three-Dimensional Deep Penetration Problems.
Slater, C.O.; Robinson, J.C.
March, 1973
Director, USA Ballistic Research Lab., ATTN: AMXBR-XM-SE,
Aberdeen Proving Ground, Maryland 21005
- CEA-CONF-2190 (In French); CONF-721018-20 (In French)
Behavior under Neutron Irradiation of Materials
Intended for the Protection Shielding of Nuclear Reactors.
Yvars, M.; Degas, P.; Quetier, M.; Rousseau, G.
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- CEA-CONF-2191 (In French); CONF-721018-19 (In French)
Particular Properties of Protective Shielding Concrete
Influencing the Attenuation Process of Radiations.
Dubois, P.
No Date
Dep., NTIS (U.S. Sales Only)
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Present Day Methods of Testing of Shielding Against
Gamma Rays.
Fourcade-Cancelle, N.; Papet, L.
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Problems in Construction of Radiation Protection.
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- CONF-700450
Problems in Construction of Radiation Protection. V.
Reports Presented at a Meeting on Radiation Protection,
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- CONF-730302-10
Gamma-Ray Spectrum Catalogue: A User Data File.
Heath, R.L.
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- DP-1322
Neutron Spectra from Californium-252 Wire Sources.
Herold, T.R.
March, 1973
Dep., NTIS

GULF-RT-A-10998
Fast Reactor Spectrum Measurements. Technical Summary
Report, February 1, 1971 - January 31, 1972.
Young, J.C.; Neill, J.M.; Verbinski, V.A.
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NTIS

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Time-Dependent Fast-Neutron and Secondary Gamma-Ray
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Three-Dimensional Analysis of the FTR Shield Mock-Up
Experiment.
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Nuclear Data Requirements for Reactor Shielding
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Compilation of (n, 2n) Cross Sections Measured at
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Beta Decay of Gaseous Fission Products.
Halbig, J.K.
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Linear Functions, Differing Weights, and Nonlinear
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Moore, R.H.
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LA-5145-MS
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Survey of Nuclear Data Use in Applied Fields.
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Neutronic Calculations in a Simulated Fusion Reactor
Blanket.
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- ORNL-RSIC-31 (Vol. 1)
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Conceptual Design of the Blanket and Shield Region
and Related Systems for a Full Scale Toroidal Fusion
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An Estimation Technique for Monte Carlo Calculations
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The ORNL Benchmark Experiment for Neutron Transport
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Analysis of a Bench-Mark Calculation of Tritium
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Integral Neutron Scattering Measurements on Iron
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The Development and Application of a Discrete
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Shield Weight Optimization.
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- ORNL-TM-4204
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Lectures on Mathematical Foundations of the Finite
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Computer Aided Decomposition of Gamma Spectra Emitted
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- RDT-E-2-4-T(4-73)
Shield Plug and Closure Cap for Penetrations: LMPER
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Division of Reactor Development and Technology (AEC),
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- RRA-T7211
Radiation Distributions for Shielding Calculations.
Mooney, L.G.
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Radiation Research Associates, Inc., 3550 Hulen St.,
Fort Worth, Texas 76107

- RT/FI-(72)33
Solutions to the Monoenergetic Neutron Transport
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- RT/FI-(72)34
Transport Theory of Monoenergetic Neutrons in Finite
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Algorithms for Discrete Geometry in Monte Carlo
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Nomograms for the Computation of the Atmospheric
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Handmade Concrete Containers for Radioactive Waste.
Relations Between Intensity of Exposure at Contact,
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Reduction of Neutron Damage in Silicon by Thick Shields
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Neutron Inelastic Scattering, 1972. Proceedings of a
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Status Report to USNDC.
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Utilization of Scientific Data Bases at LLL Using the
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Atlas of Photoneutron Cross Sections Obtained with
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Perturbation Calculations in Transport Theory Applied
to Local Perturbations.
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Empirical Formula, Continucus in Field Parameters and
Depth, for the Axial Dose for 60-Co Gamma Radiation.
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Measurements of Induced Radioactivity in Dust Around
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Fluence and Dose Equivalent Determination in a
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Isotropic Incident Flux in Monte Carlo Calculations.
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The Experience of the Central Electricity Generating
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Solution of Stationary Radiation Transport Problem for
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Gamma-Ray Attenuation in Basement Ceilings.
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Evaluation of the Second-Order Perturbation Terms by
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COMPUTER CODES LITERATURE

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The RAFFLE General Purpose Monte Carlo Code
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Hanford Engineering Dev. Lab., Richland, Wash.
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MULTI: Multigroup or Multipoint P-3 Programme for
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Matausek, M.V.
Institut za Nuklearne Nauke Boris Kidric,
Belgrade, Yugoslavia
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Formation by Reactor Irradiation Under Changing
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An ICL Version of the O5R Program System
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Central Research Institute for Physics,
Budapest, Hungary
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EXOTIC: A Computer Code for Energy Levels of
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Los Alamos Scientific Lab., Los Alamos, N. Mex.
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- LA-5157-MS March 1973 MCG;MCP
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DERTS, Toulouse, France
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A Computer Code Utilizing ANISN Radiation Transport
Calculations for Cross Section Sensitivity Analysis
Bartine, D.E.; Mynatt, F.R.; Oblow, E.M.
Oak Ridge National Lab., Oak Ridge, Tenn.
FORTRAN IV; IBM 360
AVAIL: NTIS
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The Development and Application of a Discrete
Ordinates Adjoint Difference Method for One-
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Oak Ridge National Lab., Oak Ridge, Tenn.
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AVAIL: NTIS
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Oak Ridge National Lab., Oak Ridge, Tenn.
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AVAIL: NTIS
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AVAIL: NTIS
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Kirkegaard, P.
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B-6700
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Central Electricity Generating Board, Berkeley, England