

# RSIC Newsletter



RADIATION SHIELDING INFORMATION CENTER

## OAK RIDGE NATIONAL LABORATORY

OPERATED BY UNION CARBIDE CORPORATION • FOR THE U.S. ATOMIC ENERGY COMMISSION

POST OFFICE BOX X •  
OAK RIDGE, TENNESSEE 37831

No. 42

May, 1968

*The more extensive a man's knowledge of what has been done,  
the greater will be his power of knowing what to do... Disraeli*

### WEAPONS RADIATION SHIELDING HANDBOOK QUESTIONNAIRE

RSIC has now distributed 3 chapters of the Weapons Radiation Shielding Handbook. We would like an indication as to the desirability of continuing to issue subsequent chapters as they become available and an evaluation of their worth to the shielding community. A questionnaire in this regard is appended at the end of this newsletter. If you have received these chapters, we would greatly appreciate your taking the time to fill it out and mail it to us, anonymously if you wish. Additional comments are also invited.

It is anticipated that the results of the questionnaire will affect future decisions regarding the preparation and distribution of similar documents.

### FOR CCC-17/05R USERS

An edit routine for an 05R Collision Tape, compatible with FORTRAN-63 and FORTRAN-IV, is now available. Also added to the code package are edit routines for the 05R Cross Section tapes of the type generated by Code 6, Code 7, and Code 8 on either the CDC-1604 or the IBM 360/75.

### AMC CODE PACKAGE UPDATE

The AMC Code, CCC-90, has been updated. The epicadmium and the fast-neutron albedo data, formerly supplied in the form of binary files, has been converted to BCD format. The AMC Code has been modified to read these BCD data. The master tape now contains 8 files of information, the first and second files containing the epicadmium data and the fast-neutron data, respectively.

### CORRECTIONS TO CODE PACKAGE

Changes to CCC-98/FASTER code package have been received from T. M. Jordan which include corrections and changes required for IBM 360/65 compatibility. Users of the program may secure a statement of the changes from the RSIC Codes Coordinator upon request.

### CCC-48/QAD DOCUMENTATION UPDATE

Dr. J. Robert Streetman of the N Division, LASL, has added a Supplement to LA-3573 report which describes QAD V, the Program for Calculation of Volume Heating. This code, involving two-dimensional kernel integration, is already a part of the CCC-48 code package. The new Supplement is available from RSIC on request and from normal document distribution centers. It is listed as LA-3573 Supplement. It will obsolete the informal documentation previously distributed.

### PERSONAL ITEMS

We incorrectly noted in March that Charles (Chuck) Garrett is now at Fort Worth. He is with Radiation Research Associates, as we mentioned, but is Project Manager, Information Systems Group, Bethesda, Maryland. Sorry, Chuck.

G. W. (Gene) Ray has left Aerospace and is now Chief, Strategic Studies Group, Operations Analysis Office, Headquarters, U.S. Air Force.

James Baran has finished his doctoral work at KSU and is now at Los Alamos.

### LETTER TO THE EDITOR

Dear Codes Coordinator:

*In your published distribution of abstracts, ORNL-RSIC-13, you asked that errors, omissions, etc. be called to your attention. In view of this I would like to remind you that the term "code package" has been used in Argonne Code Center publications dating back to 1960 to describe the collection of materials included in your description. While I heartily endorse your use of the term (usage breeds acceptance), I do question its "coining" by RSIC.*

*(Mrs.) Margaret Butler  
Applied Mathematics Div.  
Argonne National Laboratory*

RSIC Reply:

Webster's definition of to "coin" a word or phrase is: to make; fabricate; invent. Since RSIC neither made, fabricated, nor invented the term "code package", but rather used it in a special sense, we stand corrected. If the word "coined" is changed to the word "used" in the first sentence of the Preface it will be more nearly correct.

#### VISITORS TO RSIC

Visitors to RSIC during the month of April are: Charles Esienhauer, National Bureau of Standards, Washington, D. C.; A. B. Chilton, University of Illinois, Urbana, Illinois; Jimmy D. Matthews, Health Physicist, Ohio University, Athens, Ohio; Capt. Enz, DASA, Washington, D. C.; Terry W. Smith, Henry T. Smith, Ric Burn, Brown Engineering, Huntsville, Ala.; Janet Lacetera, Lt. Anthony Buhl, Edgewood Arsenal, Maryland.

#### MAY ACCESSION LIST OF LITERATURE

The RSIC is now aware of the literature cited in the following list. This literature has either been obtained by RSIC or has been placed on order. When received, this material will be examined and assigned to various files if suitable for our information system. The accession list is divided into three fields of (1) reactor and weapons shielding, (2) space and accelerator shielding, and (3) shielding computer codes. These titles are announced before processing and indexing so that there will be no delay and can serve as a prompt announcement of current literature.

RSIC is not a documentation center. Copies of the literature cited must generally be obtained from the author or from a documentation center such as the Clearinghouse for Federal Scientific and Technical Information, Springfield, Virginia 22151.

RSIC maintains a microfiche file of literature entered into its information system. Computer searches of this system (which produces a special bibliography) and duplicate microfiche copies of literature in our file are available upon request. Naturally we cannot supply copies of literature which is copyrighted (such as books or journal articles) or whose distribution is restricted. Neither service is yet available for the codes literature.

#### REACTOR AND WEAPONS SHIELDING

AD-647165 (DASA-1820-3)

Time Dependent Neutron Transport from a Point Isotropic source at an Altitude of (15.240Km) 50,000 Ft.  
J. P. George, A. Lavagnino  
November 17, 1966

AD-662059 (SN-255, N68-15684)

A Method for Evaluation of Radiation Shielding Design  
Final Report  
M. A. Suarez  
January 31, 1967

AECL-2793

Tables of Beta Dose Distributions  
William G. Cross  
November 1967

AERE-R-5086

Yields from Neutron Induced Fission  
I. F. Croall  
November (Reprinted January, 1968), 1965

AERE-R-5679

The Point Source Method of Dose Calculation in Media of Varying  
Density with Special Reference to Beta Particles  
W. J. Whitehouse  
January, 1968

AFWL-TR-66-111

Summary of Neutron and Gamma Dosimetry Techniques  
H. M. Murphy

EGG-1183-1334

Experimental Measurement of Fallout Protection Provided by Trans-  
portation Vehicles  
R. L. Summers  
June 30, 1967

LA-3839

A Relativity Notebook for Monte Carlo Practice  
C. J. Everett  
February 13, 1968

LA-DC-8660 (CONF-670932-2)

Design and Performance of an Open-Well Shield for High-Power Pro-  
pulsion Reactor Testing  
Glen A. Graves  
1967

NAA-SR-Memo-12586

A Design Study for Fast Flux Test Facility Secondary Gamma Shield  
J. V. Facha, J. G. Roberts  
December 20, 1967

NDL-TR-89-I

Neutron and Gamma Ray Production Cross Sections for Sodium,  
Magnesium, Chlorine, Potassium, and Calcium

Part I: General Information and Data Format, M. K. Drake, J. D.  
Garrison, et al.

Part II: Sodium, J. D. Garrison, M. K. Drake

Part III: Magnesium, M. K. Drake

Part IV: Chlorine, M. S. Allen, M. K. Drake

Part V: Potassium, M. K. Drake

Part VI: Calcium, M. K. Drake

November, 1967

NSJ-tr-93 (In English)

Analysis of Photoneutron Flux Distribution in Water-Shielded Re-  
actors

Translated from, Nippon Genshiryoku Gakkaishi, 8, 12-15

Japanese

Takashi Tagami, Mitsuyuki Kitazume

1966

ORNL-tr-1849, Translated from EUR-3472.d (German)

The Calculation of Place-Dependent Neutron Spectra in Water-Iron-  
Water Shielding Assemblies and the Relations of the Diffusion

R. Fiebig

USNCEL-TR-R565

Fast-Neutron Streaming Through Concrete Ducts

Y. T. Song

February, 1968

USNRDL-TR-68-3

Significance of Fallout Ingress into Shelters

H. Lacayo, Jr., M. A. Sullivan

October 19, 1967

Am. Ind. Hyg. Assoc. J., 29, 1, p.94-100 (1968)

Broad-Beam Gamma Attenuation in Thin Absorbers

E. B. Stewart, J. O. Ledbette

Applied Scientific Research, 16, 4, 280-287, (December, 1967)

Scattering of Directed Radiation by a Cylinder

N. R. Zitron, J. Davis

Atomkernenergie, 12: 409-14, (1967) (In German)

Measured Intensity Distributions of Neutrons and Gamma-Radiation  
of a Plane Parallel Beam and Relaxation Lengths as Functions of  
the Volume Percentage of the Metal Component in the Shield

G. Thuro, M. Heske, G. Richter, C. Schnier, I. Thuro

Atomkernenergie 13: 1-7, Jan-Feb, 1968

Semianalytical Methods for Neutron Transport Theory in Plane Geometry  
D. Bünemann, et al.

Brit. J. Appl. Phys., Vol. 1, 137-47, February 1968

The Thermal-Neutron Milne Problem with Capture  
M. J. Lancefield, et al.

Health Phys., 13, 1175-86, November 1967

Dose Distributions as a Function of LET and Measurements of QF around the BNL Medical Research Reactor  
L. F. Phillips, E. D. Scalsky, R. J. Champagne

Health Phys. 14, (4), 331, 1968

On Buried-Source Model for Computing Fallout Ground Roughness Effects  
R. L. French

Izy. Vyss. Uch. Zaved. Fiz., 5, 147-9, 1967 (In Russian)

On the Transition Effect Theory for Gamma Radiation  
V. S. Galishev

J. of Nuclear Medicine, MIRN, Sup. No. 1, p. 7-14, February 1968  
Phamphlet No. 1, Medical Internal Radiation Dose Committee

A Schema for Absorbed-Dose Calculations for Biologically-Distributed Radionuclides  
Robert Loevinger, Mones Berman

J. of Nuclear Medicine, MIRN, Sup. No. 1, p. 15-25, February, 1968  
Phamphlet No. 2

Energy Deposition in Water by Photons from a Point Isotropic Sources  
Martin J. Berger

J. of Nuclear Medicine, MIRN, Sup. No. 1, p. 27-39, February, 1968  
Phamphlet No. 3

Absorbed Fractions for Photon Dosimetry  
G. L. Brownell, W. H. Ellett, A. R. Reddy

J. Nucl. Med. Symposium, 17, 1968

Energy Deposition in Water by Photons from Point Isotropic Sources  
M. J. Berger

J. Phys. A. 1 (2), 256-, 1968

Photoelectric Cross Sections of Gamma Rays in Al, Cu, Sn, W, Au, and Pb in Energy Region 50-208, keV  
K. Parthasa

- Mater. Res. Std. 8, 3, p. 19-24, 1968  
Attenuation of X-Rays and Gamma Rays in Concrete  
B. E. Foster
- Nippon Genshiryoku Gakkaishi 9, 597-603, 1967 ( In Japanese)  
Gamma-Ray Leakage through a Slit in Lead Shield. I.  
Shun-ichi Miyasaka, Yoshihiko Kanemori, Yutaka Fukushima,  
Takeshi Yamada
- Nucl. Instr. Methods 60, 1, p. 4-6, 1968  
Monte Carlo Calculation of Source-to-Detector Geometry, II  
I. R. Williams
- Nucl. Eng. Design 6, 251-263, 1967  
A Method to Estimate the Water Content of Concrete Shields  
H. K. Hilsdorf
- Nucl. Sci. Eng. 31, 3, 492-99, March 1968  
The Application of Track-Length Distribution Biasing in Monte Carlo  
Deep-Penetration Calculations  
R. H. Karcher, R. C. Erdmann, O. C. Baldonado
- Nucl. Sci. Eng., 31, 3, 500-504, March, 1968  
The use of Self-Optimized Exponential Biasing in Obtaining Monte  
Carlo Estimates of Transmission Probabilities  
L. B. Levitt
- Nucl. Sci. Eng. 32, 1, 62-75, April, 1968 (ORNL-TM-1867)  
A Method for the Calculation of Neutron-Capture Gamma-Ray Spectra  
K. J. Yost
- Nucl. Sci. Eng. 32, 1, 76-81, April, 1968  
Mathematical Verification of a Certain Monte Carlo Sampling Technique  
and Applications of the Technique to Radiation Transport Problems.  
W. A. Coleman
- Nucl. Sci. Eng. 32, 1, 120-130, April, 1968  
Biorthogonal Angular Polynomial Expansion of the Two-Dimensional  
Transport Equation  
K. D. Lathrop, N. S. Demuth
- Nucleonik 10, p. 283-7, January 1968 (In German)  
Neutron Spectrum of Am-Be Source  
H. B. Greiss

Phys. Med. Biol., 13 (2), 219 , 1968

An Assessment of Monte Carlo Calculations to Determine Gamma Ray  
Dose from Internal Emitters  
W. H. Ellett, G. L. Brownell, A. R. Reddy

Proc. Phys. Soc. London, 92, 618, 1967

Photon Attenuation Cross Section of Uranium  
J. L. Perkin, A. C. Douglas

Rentgenol. Radiol., 5, 212-17, 1966 (In German)

Application of Computers in the Determination of the Dose in Radio-  
Therapy  
J. Richter, P. Penchev

Soviet J. At. Energy, English Transl., 22, 5, 505, May 1967

The Scattering of 0.511 MeV Gamma Rays by Shielding Barriers  
B. M. Skvortsov, V. V. Chernyakhovski, L. S. Sheiman, D. B. Pozdneev

Soviet J. At. Energy, English Transl. 22, 5, 507-508, May 1967

The Spatial Distribution of Backscattered Gamma Radiation from Sources  
on the Surface of a Reflector  
N. F. Andrushin, B. P. Bulatov

Soviet J. At. Energy, English Transl, 22, 5, 522-523, May, 1967

The Spatial Distribution of Scattered Gamma Radiation in a Cavity  
for a Collimated Primary Source  
N. F. Andrushin, B. P. Bulatov

BOOK

Principles of Radiation Protection  
K. Z. Morgan, J. E. Turner (Editors)  
A Textbook of Health Physics  
New York, John Wiley and Sons, Inc. Publishers  
1967

BOOK

The Chemistry and Physics of High Energy Reactions  
E. J. Henley, E. R. Johnson

BOOK

Background, Shielding, and Collimation  
W. E. Kreger, R. L. Mather  
S. M. Shafroth, (Editor)  
New York, Gordon and Breach Science Publishers  
1967



BOOK

USA Standard Glossary of Terms in Nuclear Science and Technology  
(USAS N1.1-1967)  
USA Standards Institute  
10 E. 40th St.  
New York, New York 10016

THESIS

Backscatter of Normally Incident Gamma Protons from Semi-Infinite  
Media of Varying Atomic Number  
Julian Jack Steyn  
Toronto, University of Toronto

COMPUTER CODES LITERATURE

Risö-M-663	December 1967	PRIGAM SEGAM I and II REMTHERM MC4 REMDIFF
User's Manual for the AEK Shielding Programs P-18, 19, 20, 63, 220, 374 By Peter Kirkegaard ALGOL for GIER - ALGOL for IBM 7094 - FORTRAN IV for IBM 7094		
UJV 1753 (ORNL-tr-1844)	February 1967	NEFDORN
Computer Program for the Calculation of Neutron Fluxes, Dose Rates and Heat Sources in the Shielding of a Nuclear Reactor by J. Burian ALGOL for GIER		
UJV 1755 (ORNL-tr-1845)	February 1967	GAMRAT
Calculation of the Gamma Flux Dose Rate and Heat Source in the Shielding of a Nuclear Reactor by J. Rataj ALGOL for GIER		
LA-3573 Supplement	April 1968	QAD V
The Program QAD V for Calculation of Volume Heating by J. R. Streetman FORTRAN IV, IBM 7090, 7094		

LMSC-5234

February 1968

AIRTRANS

Utilization Manual - Computer Program AIRTRANS (a time-energy dependent, 3-dimensional, Monte Carlo system in a variable density atmosphere; manual written at Lockheed Missiles & Space Division.)  
FORTRAN IV, UNIVAC 1108

AE-FFN-66

October 1967

MULTPOL

Finite-Geometry and Polarized Multiple-Scattering Corrections of Experimental Fast-Neutron Polarization Data by means of Monte Carlo Methods  
By O. Aspelund and B. Gustafsson  
FORTRAN, IBM

A QUESTIONNAIRE ON THE WEAPONS RADIATION SHIELDING HANDBOOK

The following Chapters of the Weapons Radiation Shielding Handbook have been distributed by RSIC:

- Chapter 3: Methods for Calculating Neutron and Gamma-Ray Attenuation
- Chapter 4: Neutron and Gamma-Ray Albedos
- Chapter 5: Methods for Calculating Effects of Ducts, Accessways, and Holes in Radiation Shields

Please indicate your feelings as to the value of each chapter by circling the appropriate descriptive words in the following list. Also feel free to add any additional comments.

1. Would you like to receive subsequent chapters and revisions?
- |     |    |           |
|-----|----|-----------|
| YES | NO | Undecided |
|-----|----|-----------|

2. In general, do you think the handbook will be useful to the shielding community?
- |     |    |           |
|-----|----|-----------|
| YES | NO | Undecided |
|-----|----|-----------|

3. What do you think of the coverage?
- |            |           |      |      |      |           |
|------------|-----------|------|------|------|-----------|
| Chapter 3: | Excellent | good | fair | poor | undecided |
| Chapter 4: | "         | "    | "    | "    | "         |
| Chapter 5: | "         | "    | "    | "    | "         |

4. What is your opinion in regard to clarity and manner of presentation?
- |            |           |      |      |      |           |
|------------|-----------|------|------|------|-----------|
| Chapter 3: | Excellent | good | fair | poor | undecided |
| Chapter 4: | "         | "    | "    | "    | "         |
| Chapter 5: | "         | "    | "    | "    | "         |

5. Have you actually used the material presented either for specific or general information?
- |            |     |    |
|------------|-----|----|
| Chapter 3: | YES | NO |
| Chapter 4: | "   | "  |
| Chapter 5: | "   | "  |

6. Have you placed the Chapters on your reference shelf for future use?
- |            |     |    |
|------------|-----|----|
| Chapter 3: | YES | NO |
| Chapter 4: | "   | "  |
| Chapter 5: | "   | "  |

Please mail this completed questionnaire to:

Radiation Shielding Information Center  
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 Oak Ridge, Tennessee 37830